			Authors, Compilers, or	Date (M/D/Y),	Location, Site, or		1	Format/Copy/C	Ensure	
Index #	Document Number	Document Title	Editors	(M/Y) or (Y)	Company	Status Markings	Keywords	ondition	Availability	Notes
									Codes:	
									N=ORNL/NCS	;
I									Y=NTIS/	
									OSTI, J-	
									5700=	
									Johnson	
									Collection-	
									ORNL-Bldg-	
									5700; T-	
									CSIRC=	ORNL/ Nuclear Criticality Safety
									Thomas	Group,865-574-1931; NTIS/OSTI is the
									Collection,	DOE Information
									LANL/CSIRC.	Bridge:http://www.osti.gov/bridge/
			J. W. Morfitt, R. L. Murray,			Secret, declassified	computational method/data	report, original,		Early hand calculation methods to
1	A-7.390.22	Critical Conditions in Cylindrical Vessels	G. W. Schmidt	01/28/1947	Y-12	12/12/1956	(1)	good	N	estimate process limits for HEU solution
										Use of early hand calculation methods to
		Calculation of Critical Conditions for Uranyl				Secret, declassified	computational method/data	report, original,		predict critical conditions. Done to assist
2	A-7.390.25	Fluoride Solutions	R. L. Murray	03/05/1947	Y-12	11/18/1957	(1), experiment plan/design	good	N	design of K-343 solution experiments.
		Fabrication of Zero Power Reactor Fuel Elements						CSIRC/Electroni	T-CSIRC, Vol-	Early work with U-233, Available through
3	A-2489	Containing 233U3O8 Powder		4/1/44	ORNL	Unknown	U-233 Fabrication	с	3B	CSIRC/Thomas CD Vol 3B
										Contains plans for solution preparation,
		Outline of Experiments for the Determination of								experiment apparatus, and experiment
		the Critical Mass of Uranium in Aqueous	C. Beck, A. D. Callihan, R.			Secret, declassified		report, original,		facility. Potentially useful for
4	A-3683	Solutions of UO2F2	L. Murray	01/20/1947	ORNL	10/25/1957	experiment plan/design	good	Y	benchmarking of K-343 experiments.
										Plans for a general-purpose split table
			F. E. Crever, G. Dessauer,							experimental assembly to simulate a
			W. H. Ellis, L. L. German, J.							breeding power reactor with HEU fuel,
			H. Germer, D. Jacob, F. G.							Be moderator, Na coolant, and DU
		Feasibility Report for the Zero Power Pile at the	LaViolette, H. Schultz, T. M			Secret, declassified		report, original,		reflector/breeding blanket.
5	A-4207	Sacandaga Laboratory	.Snyder, V. C. Wilson	05/16/1947	KAPL	10/25/1957	experiment plan/design	good	N	Intermediate-energy spectrum.
										Material buckling measurements by the
										exponential method at four BeO/U-235
1		Buckling and Integral Measurements in	P. Duerden, D. B.		Australian Atomic			report, original,		ratios from ~ 1500 to ~8800. U-235
6	AAEC/E123	U235/BeO Sub-Critical Assemblies	McCullouch, E. Brittliff	07/1964	Energy Commission	Unclassified	experimental criticality data	good	N	enrichment of 89.4%.
										Similar to AAEC/E123, except that the
										BeO moderated is mixed with either
		Buckling and Integral Spectrum Measurements								natural uranium oxide or thorium oxide.
		in U235 Fuelled Sub-Critical Assemblies	D. B. McCullouch, P.		Australian Atomic			report, original,		BeO/U-235 ratios ranged from ~ 1500 to
7	AAEC/E146	Moderated by BeO/Fertile Material Mixtures	Duerden, E. Brittliff	12/1965	Energy Commission	[None]	experimental criticality data	good	N	~5700.
										Monograph of lecture series provided to
										physicists, chemists, and engineers
							computational method/data	report, original,		during a 1-year assignment at ORNL
8	AECD-2201	Elementary Pile Theory	H. Soodak, E. C. Campbell	08/04/1948	ORNL	Declassified	(1)	good	N	(Clinton Laboratories) during 1946-1947.
				/			computational method/data	report, original,		
9	AECD-3740	Interaction of Enriched Uranium Assemblies	H. F. Henry	11/23/1949	K-25	Unclassified	(1)	good	N	Genesis of solid angle methodology.
										Summary of experimental results for pair
										of identical cylinders of uranyl nitrate
			A. V. Kamaev, B. G.							solution [U(90)], air-spaced. Also, pairs
1			1							of U(2) and U(10) heterogeneous metal
1		Experimental Investigation of Effects of	Dubovskii, V. V. Vavilov, G.		Buccia (UCCD)			roport com:		
10	AFC += 4700	Experimental Investigation of Effects of	A. Popov, Yu. D.	1000	Russia (USSR);	[None]	oversemental cuts lite and a	report, copy,	, .	assemblies, spaced in water. Details of
10	AEC-tr-4708	Interaction of Two Subcritical Reactors	Palamarchuk, S. P. Ivanov G. I. Marchuk, G. A.	1960	unspecified site	[None]	experimental criticality data	fair	N	configurations are not provided.  Translation by AEC. Hand methods to
			Ilyasova, V. E. Kolesov, V.							extrapolate critical mass mass data to
		Critical Masses of Agueous Mixtures of Uranium			Russia (USSR);		computational method/data	report conv		differing enrichments, using data for
11	AEC-tr-4712	4	P. Kochergin, L. I. Kuznetsova	1960	unspecified site	[None]		report, copy, fair	N	
11	ALC-U-4/12	and Plutonium Compounds	Nuziletsuva	1200	unspecified site	[None]	(1)	Iall	i in	water-moderated UO <sub>2</sub> or UO <sub>2</sub> +PuO <sub>2</sub> .

								<del>,</del>		
			G. I. Marchuk, G. A.							
			Ilyasova, V. E. Kolesov, V.							
			P. Kochergin, L. I.							
			Kuznetsova, E. I.		Russia (USSR);		computational method/data	report, copy,		Similar to AEC-tr-2712, with U-Be as the
12	AEC-tr-4713	Critical Massas of Hranium Bondlium Bonstons	1	1960		[None]		fair	N	
12	AEC-tr-4/13	Critical Masses of Uranium-Beryllium Reactors	Pogudalina	1960	unspecified site	[None]	(1)	Tair	IN	fissile material and Be as the reflector.
		Approximate Method of Calculation of Critical								
		Masses of Spherical Reactors with Infinite	G. I. Marchuk, V. P.		Russia (USSR);		computational method/data	report, copy,		Similar to AEC-tr-2712, various fissile
13	AEC-tr-4714	Reflector	Kochergin	1960	unspecified site	[None]	(1)	fair	N	mixture and reflector conditions.
			G. I. Marchuk, G. A.					-		
			Ilyasova, V. E. Kolesov, V.							
			1							
			P. Kochergin, L. I.							
			Kuznetsova, E. I.		Russia (USSR);		computational method/data	report, copy,		Similar to AEC-tr-2712, with U-C as the
14	AEC-tr-4715	Critical Masses of Uranium-Graphite Reactors	Pogudalina	1960	unspecified site	[None]	(1)	fair	N	fissile material and C as the reflector.
					· · · · · · · · · · · · · · · · · · ·	· · · · · · · · · · · · · · · · · · ·		conference		Early Pu <sup>238</sup> cross section measurements,
								1 :		1 7
		220						paper, copy,		0.4 to 1.4 MeV. Copy printed from
15	AED-Conf-63-048-54	Fission Cross Section of Pu <sup>238</sup>	D. K. Butler, R. K. Sjoblom	04/22/1963	ANL	[None]	nuclear data/measurement	fair	N	microfiche.
										Overview of many steps for chemical
										processing of U and Pu, discussion of
			K D D Jaharan D E		Hammell A E D E					· · · · · · · · · · · · · · · · · ·
			K. D. B. Johnson, R. F.		Harwell A.E.R.E.			report, original,		equipment design areas where
16	AERE-M 601	Processing in Limited Geometries - Part One	Taylor	01/07/1960	(U.K.A.E.A.)	©, Official Use Only	equipment/process design	good	Υ	geometric control may be applied.
								1		Experiments with U233 solution and with
								1		
										intermediate-enrichment U <sup>235</sup> solution
										[U(44.6)], various concentrations, 12-
		Critical Assemblies of Aqueous Uranyl Fluoride								inch diameter reactor, unreflected,
		1	W. C. Clarka C. C. Harton		Hamuell A F D F			report, original,		water-reflected and cadmium-water
		Solutions Part I Experimental Techniques and	W. G. Clarke, C. C. Horton,		Harwell A.E.R.E.			1		
17	A.E.R.E. R/R 2051	Results	M. F. Smith	09/20/1956	(U.K.A.E.A.)	Unclassified	experimental criticality data	good	Y	reflected conditions.
										Configurations are highly subcritical and
		The Interpretation of Approach-to-Critical								consist of 1.2-inch diameter U(1.0),
		Experiments with Application to Organic Liquid	L. G. Sanders, A. K.		Harwell A.E.R.E.			report, original,		U(1.3) and U(1.6) rod lattices with
40	4505 D /D 2250	1	· ·	42/4057		0 11 1 15 1		1		
18	AERE R/R 2358	Moderated Systems	McCracken	12/1957	(U.K.A.E.A.)	©, Unclassified	experimental criticality data	good	N	organic liquid moderator.
										Experiments with 44.6% enrichment U <sup>233</sup>
			J. R. Harrison, M. F. Smith,							solution in D₂O at various
		Critical Assemblies with Heavy Water Solutions	W. G. Clarke, A. M. Mills,		Harwell A.E.R.E.			report, original,		concentrations, with graphite reflector
10	A EDE D /D 2702	-		44/4050		@ !!!-:f:!			Υ	
19	AERE R/R 2703	of Uranyl Fluoride (H.A.Z.E.L.) Part 2 Physics	Miss J. A. Dyson	11/1958	(U.K.A.E.A.)	©, Unclassified	experimental criticality data	good		about a 2-ft diameter cylindrical tank.
										Two-group diffusion theory analysis of
		Critical Assemblies with Heavy Water Solutions								experiments of AERE R/R 2703. Also,
		of Uranyl Fluoride (H.A.Z.E.L.) Part 3 Theoretical	C. Carter, P. K. H. Lang, G.		Harwell A.E.R.E.		computational method/data	report, original,		early computer modeling with
20	AERE R/R 2731	Analysis	Myatt	07/1959	(U.K.A.E.A.)	©, Unclassified	(1)	good	N	multigroup methods (6-groups).
	ACKE N/K 2751	Principolo	wyatt	07/1555	(0.10.70.2.70.)	e, onclassifica		Bood		manigroup methods (o groups).
										Document is comparable to early U. S.
										criticality safety/data handbooks, with
										somewhat greater focus on criticality
			C. M. Nicholls, E. R.		Harwell A.E.R.E.			report, original,		theory and criticality safety applications
2.5	AEDE D 204.4	Cathian lite		02/2050		@ Off:-:-! 0 :	la a a dia a a la	1	.,	
21	AERE-R 2914	Criticality	Woodcock, A. H. Gillieson	03/1959	(U.K.A.E.A.)	©, Official Use Only	handbook	good	Υ	than data.  Collected results from critical
										experiments; Chapter 5 covers single
								1		units of hydrogen-moderated <sup>235</sup> U, and
								1		
		Handbook of Experimental Criticality Data PART						report, original,		Chapter 6 covers singlue units of
22	AHSB (S) HANDBOOK 5	2 - Chapters 5 and 6	F. Abbey	1968	U.K.A.E.A	©, Unclassified	handbook	good	Υ	hydrogen-moderated Pu
										Computations are for idealized 1-D
		Critical Assemblies of Infinite Slabs of Highly					computational method/data	report, original,		configurations, very little information on
22	ALICO Descrit 47	- 1	E B Weeder !	07/4050		@ UI 'C		1 1		1 -
23	AHSB Report 17	Enriched Uranium and Water	E. R. Woodcock	07/1959	U.K.A.E.A	©, Unclassified	(1)	good	N	computer code method is provided.
								1		Uses matrix algebra to determine if an
								1		array of units is subcritical, critical, or
								1		supercritical; obsolete methodology;
								1		- · · · · · · · · · · · · · · · · · · ·
		Criticality of Interacting Arrays of Fissile Material					computational method/data	report, original,		does not directly result in k <sub>eff</sub>
24	AHSB (S) R28	Part 1: General Theory	D. C. Dowson	10/1961	U.K.A.E.A	©, Unclassified	(1)	good	N	predictions.
										Elaboration of AHSB (s) R28 techniques
								1		for special conditions. Requires
										knowledge of "surface multiplication" of
		Criticality of Interacting Arrays of Fissile Material					computational method/data	report, original,		units and solid-angle. Obsolete
25	AHSB (S) R29	Part 2: Unreflected Air-Spaced Arrays of Spheres	D. C. Dowson.F. Abbev	10/1961	U.K.A.E.A	©, Unclassified	(1)	good	N	methodology
		, and a second s		.,		,	p. 7	U		

		Criticality of Interacting Arrays of Fissile Material					computational method/data	report, original,		Like AHSB (S) R29, except for arrays with
26	AHSB (S) R30	Part 3: Reflected Air-Spaced Arrays of Spheres	D. C. Dowson	11/1962	U.K.A.E.A	©, Unclassified	(1)	good	N	reflectors.
		Review of U.K.A.E.A. Criticality Detection and		,		, , , , , , , , , , , , , , , , , , , ,	(-)	8000		Addresses basic design issues for
		Alarm Systems 1963/64 Part 1: Provision and	K. J. Aspinall and J. T.					report, original,		criticality alarm systems and evacuation
27	AHSB (S) R92	Design Principles	Daniels	1965	U.K.A.E.A	©, Unclassified	criticality accident	good	Υ	zone selection
	A113B (3) 1(32	Review of U.K.A.E.A. Criticality Detection and	Daniels	1303	U.K.A.L.A	e, onclassified	criticality accident	good		Zone selection
		Alarm Systems 1963/64 Part 1: Provision and						1		
		1	K. J. Aspinall and J. T.					roport original		Modifies some recommendations of
20	No	Design Principles Amendment of Proposals	-	44/4005	111/ 4 5 4	[NI 1		report, original,	Υ	
28	No report number	Concerning Plutonium Systems	Daniels	11/1965	U.K.A.E.A	[None]	criticality accident	good	Y	AHSB (S) R92
		Operating Instructions for the Monte Carlo						report,		Document copy is poor but legible;
		Neutronics Program GEM1 and the Associated	P. J. Hemmings, T. C.				computational method/data	mimeograph,		numerous handwritten markings are
29	A.H.S.B.(S)M.126	Programs CHECK and POND	Longworth	11/1964	U.K.A.E.A	NOT FOR PUBLICATION	(2)	marginal	N	present throughout.
					National Reactor					
		A Summary of ETR Critical Facility Safety Analysis	- '		Testing Station, Idaho			report, original,		
30	ANCR-1002	Information	Kaufman, J. W. Henscheid	07/1971	Falls ID	U	experiment safety analysis	good	N	
								1		Evaluates the practically of a pulsed-
					National Reactor			1		fusion source of neutrons for time of
		The Potential of a Laser-Induced Fusion Device			Testing Station, Idaho			report, original,		flight measurements. Errata sheet
31	ANCR-1034	as a Thermal Neutron Source	R. M. Brugger	11/1971	Falls ID	[None]		good	N	included.
										Provides theoretical and experimental
								1		measurements of self-shielding in
					National Reactor			1		stacked gold foils. Application is to
					Testing Station, Idaho			report, original,		measure neutron energy spectra,
32	ANCR-1066	Self-Shielding in Stacked Foils	R. G. Nisle, Y. D. Harker	11/1972	Falls ID	[None]	nuclear measurement/data	good	N	particularly for epithermal reactors.
		Technical Review of ZPR-I Accidental Transient	R. O. Brittan, R. J.	· · · · · · · · · · · · · · · · · · ·			,			Physics and consequence analyses of the
		The Power Excursion, Exposures, and Clinical	Hasterlik, L. D. Marinelli, F.			Secret, declassified		report, original,		June 2, 1952 critical experiment accident
33	ANL-4971	Data	W. Thalgott	01/1953	ANL	05/15/59	criticality accident	good	Υ	with ZPR-1
						10, 10, 00		8000		Use of a pulsed neutron source to
			R. Siems, M.					report, original,		measure diffusion theory parameters (D
34	ANL-6254	A Pulsed Neutron Source	Melissaropoulos	11/1960	ANL	[None]	nuclear measurement/data	good	N	or L)
	AIVE 0254	Safety Analysis Report Argonne Fast Critical	W. Y. Kato, G. J. Fischer, L.	11/1500	AIVE	[Hone]	indical incasarcinent/data	report, original,		01 2)
35	ANL-6271	Facility (ZPR-VI)	R. Dates	12/1963	ANL	[None]	experiment safety analysis	good	N	
	AIVE-0271	Tacinty (ZFN-VI)	R. O. Brittan, B. Cerutti, H.	12/1303	ANL	[None]	experiment safety analysis	good		
			V. Lichtenberger, J. K.					1 1		
			Long, R. L. McVean, M.							
		Unanad Carlantina Banada an tha Foot Banada								
26	ANII C400	Hazard Evaluation Report on the Fast Reactor	Novick, R. Rice, F. W.	10/1061	4.511	[NI 1		report, original,		
36	ANL-6408	Zero Power Experiment ZPR-III	Thalgott	10/1961	ANL	[None]	experiment safety analysis	good	N	D
										Presentation and analysis of data
		Critical Studies of a 440-Liter Fast-Reactor Core	A. L. Hess, J. M. Gasidlo, J.							obtained from ZPR-3 Assembly 41, for
		Fueled with Uranium Enriched to 17 Percent	K. Long, P. I. Amundson,					report, original,		application of data to other experiments
37	ANL-6732	(ZPR-3 Assembly 41)	W. P. Kenney	06/1971	ANL	[None]	experiment data/analysis	good	N	and reactor design
								1 1		Focus areas: (1) re-evaluate Pu release
		Safety Analysis of the Operation of ZPR-3 with						1		fraction and consequence due to a fire,
		Fuel Loadings Up to 430 kg of Plutonium	J. K. Long, L. R. Kelman, R.							and (2) re-evaluate maximum core
		(Addendum to ANL-6504, Safety Analysis of	L. McVean, M. Novick, A.					report, original,		temperature due to Pu decay (so as to
38	ANL-7049	Plutonium Loadings in ZPR-III)	B. Shuck, F. W. Thalgott	12/1965	ANL	[None]	experiment safety analysis	good	N	not exceed Na melting temperature)
			A. R. Boynton, Q. L. Baird,							
			K. E. Plumlee, W. C.							Some lattices are considerably
			Redman, W. R. Robinson,					report, original,		undermoderated and are outside the
39	ANL-7203	High Conversion Critical Experiments	G. S. Stanford	11/1967	ANL	[None]	experimental criticality data	good	Υ	range of benchmarks in the IHECSBE.
										This document consists of replacement
		Argonne Code Center: Compilation of Program	M. K. Butler, Marianne				computational method/data	report, original,		pages for the initial issue of ANS-7411
40	ANL-7411 Supplement 1	Abstracts	Legan, L. Ranzini	10/1968	ANL	[None]	(2)	good	N	(01/1968)
	FF					† · · · · · · · · · · · · · · · · · · ·				This document consists of replacement
		Argonne Code Center: Compilation of Program	M. K. Butler, Marianne				computational method/data	report, original,		pages for the initial issue of ANS-7411
41	ANL-7411 Supplement 1	Abstracts	Legan, L. Ranzini	04/1969	ANL	[None]	(2)	good	N	(01/1968)
	, , 711 Jupplement 1	7.050.000	Benchmark Problem	0-11303	AINE	[	\ <u>-</u> '	P2000		(02,1500)
			Committee of the							This document contains three
		Argonne Code Center: Benchmark Problem Book						1		benchmarks (one a critical experiment,
		1 -						1		
		Numerical Determination of the Space, Time,	Computational Division of				computational	ronant coloinal		the other two are hypothethical) plus
42	ANI 7446	Angle, or Energy Distribution of Particles in an	the American Nuclear	02/4000		[N 1	computational method/data	report, original,		several code results for each, for use in
42	ANL-7416	Assembly	Society	02/1968	ANL	[None]	(2)	good	N	reactor physics code comparisons.

Γ			T	·	T	T		1		
										This document addresses the theory and
										experimental data analysis methods for
l		Thickness Corrections for Neutron-Activated	George S. Stanford, James					report, original,		use of gold foils in critical experiments to
43	ANL-7545	Gold Foils	H. Seckinger	02/1969	ANL	[None]	nuclear measurement/data	good	N	determine flux spectra and distribution.
45	ANL-7343	G010 1 0113	11. Seckinger	02/1303	AINL	[None]	indical measurement/data	good		The code's purpose is to predict shock
										waves, expansion, and bubble formation
l										within liquid sodium coolant due to fast
		Comparison of a Two-Dimensional								reactor transients. Code performance is
		Hydrodynamics Code (REXCO) to Excursion					computational method/data	report, original,		demonstrated using non-nuclear
44	ANL-7911	Experiments for Fast Reactor Containment	J. E. Ash, R. T. Julke	01/1972	ANL	[None]	(3)	good	N	(explosive charge) test results.
	AINL-7311	Impact Testing on Collet Assembly for Control	J. E. ASII, N. I. Juike	01/1972	General Electric (San		(3)	report, original,	IN	Results of mechanical testing on reactor
45	APED-5555	Rod Drive Mechanism 7RDB144A	J. E. Benecki	11/1967			operational/test/material data	1	N	control rod drive components
45	APED-5555	ROU DIIVE MECHANISHI 7RDB144A	J. E. Bellecki	11/1967	Jose, CA)	[None]	operational/test/material data	good	N	control rod drive components
										A brief design and operational safety
					0 151 1					handbook, with focus on enriched <sup>235</sup> U
					General Electric,					with hydrogen and/or beryllium
		Aircraft Musless Base 11: 2			Nuclear Materials and					moderation. For use with GE opprations
	ADEV 745	Aircraft Nuclear Propulsion Department Nuclear	Marilli A D	00/:	Propulsion Operation			report, original,		for fabrication of test reactor fuel for
46	APEX-715	Safety Guide	William A. Pryor	08/1961	(Cincinnati, OH)	Unclassified	handbook	good	N	aircraft propulsion systems.  Monte Carlo - computed results for
1										·
			E. R. Woodcock, J. B.		Aldermaston A.E.R.E.		computational method/data	report, original,		simple 1-D infinite slabs of <sup>235</sup> U, various
47	AWRE O-14/60	Infinite Slab Criticality Calculations	Parker	05/1960	(U.K.A.E.A.)	Unclassified	(2)	good	N	reflector conditions.
										Assemblies of U(45.5) metal, various
										reflector conditions, performed on the
										"Atlas" machine. 17 configurations total.
										(This is Reference 2 of IHECSBE report
l										IEU-MET-FAST-019, which contains 2 of
		Critical Mass Measurements with Thin Discs of	J. R. Dominey, R. C. Lane,			Unclassified, Limited		report, original,		these configurations. Also is Ref 138 of
48	AWRE NR/A-1/62	45.5% Enriched Uranium	A. F. Thomas	03/1962	(U.K.A.E.A.)	Circulation	experimental criticality data	good	Υ	LA-13860-MS.)
		The Calculation of Neutron Surface	E. D. Pendlbury, Patricia E.		Aldermaston A.E.R.E.		computational method/data	report, original,		
49	AWRE O-21/64	Multiplications of Transport Containers	Garrett	07/1964	(U.K.A.E.A.)	Unclassified	(1)	good	N	Obsolete methodology
		A Measurement of the Critical Size of a								
		Homogeneous Mixture of Plutonium and Natural			Aldermaston A.E.R.E.			report, original,		240
50	AWRE O 32/68	Uranium Oxides with Polythene	R. C. Lane, O. J. E. Perkins	07/1968	(U.K.A.E.A.)	Distribution	experimental criticality data	good	Υ	H:Pu ~ 18.6, U:Pu ~ 1, ~ 5.9% <sup>240</sup> Pu
		Measurement of the Critical Mass of 37½%								
		Enriched Uranium in Reflectors of Wood,			Aldermaston A.E.R.E.			report, original,		
51	AWRE NR 1/66	Concrete, Polyethylene and Water	R. C. Lane, O. J. E. Perkins	02/1966	(U.K.A.E.A.)	©, Unclassified	experimental criticality data	good	Υ	
					The Babcock and					Two-group diffusion theory method
			D. B. Wehmeyer, K. E.		Wilcox Company,		computational method/data	report, original,		application to available experiments,
52	BAW-150	Nuclear Safety of UO <sub>2</sub> -THO <sub>2</sub> -H <sub>2</sub> O Systems	Roach	1960	Lynchburg VA	[None]	(1), equipment/process design	good	N	plus derivation of safe operating limits
					The Babcock and			report, copy		Two-group diffusion theory incorporated
					Wilcox Company,		computational method/data	from microcard,		into a Fortran program, instructions for
53	BAW-207	SAFETY A Simplified Criticality Program	D. B. Wehmeyer	06/1963	Lynchburg VA	[None]	(1)	poor	N	use of the program
			T. C. Engelder, N. L.							Report contains detailed descriptions for
		Spectral Shift Control Reactor Basic Physics	Snidow, R. H. Clark, C. E.		The Babcock and					each assembly, some configurations are
		Program Critical Experiments on Lattices	Barksdale, R. H. Lewis, M.		Wilcox Company,			report, original,		moderated only by H <sub>2</sub> O, some
54	BAW-1231	Moderated by D <sub>2</sub> O-H <sub>2</sub> O Mixtures	N. Baldwin	12/1961	Lynchburg VA	[None]	experimental criticality data	good	Υ	configurations use U(93)O <sub>2</sub> -ThO <sub>2</sub> fuel.
		,			The Babcock and					Report contains detailed descriptions for
			R. H. Clark, M. L. Batch, T.		Wilcox Company,			report, original,		each LEU lattice assembly, burnable
55	BAW-3492-1	Lumped Burnable Poison Program - Final Report		11/1966	Lynchburg VA	[None]	experimental criticality data	good	Υ	poison rods are B <sub>4</sub> C or glass.
				, 2500	The Babcock and		- Familian Gradulty Gutu	10,22	·····	posson rous are bac or grass.
		Physics Verification Program Quarterly			Wilcox Company,			report, original,		
56	BAW-3647-1			08/1966		[None]	ovnorimental eriticality dete	1	Υ	
56	DAVV-304/-1	Technical Report No. 1 January - June 1966		08/1966	Lynchburg VA	[None]	experimental criticality data	good	Υ	
			Alton E. Klickman, George							This document addresses the theer and
										This document addresses the theory and
		A Wire Activation Technique for De	W. Cunningham, Joel W.		Dattalla Mam:!			ropost original		experimental data analysis methods for
	BMI-1086	A Wire-Activation Technique for Reactor-Flux-	Chastain, Donald L. Keller, Sherwood L. Fawcett	04/25/1956	Battelle Memorial	[None]	nuclear measurement /dst-	report, original,	N.I	use of wire in critical experiments to
57	DIAIL-TAGO	Profile Measurements	SHELMOOR F. FAWCELL	04/25/1956	Institute	[None]	nuclear measurement/data	good	N	determine flux spectra and distribution.

		1								Mathematical expressions for the <sup>238</sup> U
										resonance escape probability are
								mama ariginal		provided; an experimental measuremen
								memo, original-		
								issue copy,		approach is described and applied to the
58	BNL 404 I 16	Resonance Escape Probability Measurements	R. Sher	05/07/1956	BNL	[None]	nuclear measurement/data	good	N	BNL reactor.  Derivation of 3-group diffusion
										equations such that epithermal <sup>235</sup> U
								memo, original-		fission cannot be inferred from
		The Effect of Epi-Thermal Fissions on the					computational method/data	issue copy,		experimentally measured reactor
59	BNL 410 I 17	Neutron Cycle	R. Sher, H. Kouts	08/08/1956	BNL	[None]	(1)	good	N	parameters.
			· · · · · · · · · · · · · · · · · · ·	· · · · · · · · · · · · · · · · · · ·		· · · · · · · · · · · · · · · · · · ·		f		Demonstration of how various
										experimental measurements of reactor
		Experimental Studies of Slightly Enriched								parameters can be used to infer input
		Uranium, Water Moderated Lattices Part 1.	Herbert Kouts, Rudolph					report, original,		data/constants for reactor physics
60	BNL 486	0.600-inDiameter Rods	Sher	09/1957	BNL	[None]	experimental criticality data	good	N	calculations/power reactor design.
	BINE 480	Optical Model Analysis of Inelastic Scattering of		03/1337	DINL	[None]	experimental criticality data		IN	<del></del>
	DAN 040 (T 047)	1.	S. O. Moore, E. H.	00/4050		fa. 1	.,,,,	report, original,		Nuclear physics modeling to predict
61	BNL 818 (T-317)	Neutrons by Heavy Nuclei	Auerbach	08/1963	BNL	[None]	nuclear measurement/data	good	N	inelastic scatter
		Analysis of (n,2n) Cross Sections for Nuclei of						report, original,		Nuclear physics modeling to predict
62	BNL 897 (T-365)	Mass A > 30	S. Pearlstein	12/1964	BNL	[None]	nuclear measurement/data	good	N	(n,2n) reactions
		Optical Model Analysis of Neutron-Sodium Cross						report, original,		
63	BNL 904 (N-8)	Section in the 1-4 MeV Range	T. J. Krieger, S. Pearlstein	01/1965	BNL	[None]	nuclear measurement/data	good	N	
		Least Squares Analysis of the 2200 m/sec	Rudolph Sher, Joan					report, original,		
64	BNL 918 (T-377-92-94-2)	Parameters for U <sup>233</sup> , U <sup>235</sup> , and Pu <sup>239</sup>	Felderbaum	03/1965	BNL	[None]	nuclear measurement/data	good	N	
		The Neutron Cross Section of Sodium Below 40					1	report, original,		
65	BNL 961 (T-401)	keV	Thomas E. Stephenson	12/1965	BNL	[None]	nuclear measurement/data	good	N	
	BNE 301 (1-401)	NE V	monias E. Stephenson	12/1303	DIVE	[None]	indical measurement/data			
	DAIL 002 (T. 445)		6 B 14 1	05/4055		fa. 1		report, original,		
66	BNL 982 (T-415)	Cross Sections for Transuranium Production	S. Pearlstein	05/1966	BNL	[None]	nuclear measurement/data	good	N	
			Kenneth W. Downes,					report, original,		
67	BNL-1785	Reactivity Coefficient Measurement of Buckling	Herbert J. Kouts	03/18/1954	BNL	Unclassified	experimental criticality data	good	N	
								report, original,		
68	BNL-1992	Thermal Utilization Measurement	G. A. Price	08/19/1954	BNL	Unclassified	nuclear measurement/data	good	N	
		Buckling of a Natural Uranium Light Water						report, original,		
69	BNL-2016	Moderated Lattice	K. Downes	08/23/1954	BNL	Unclassified	experimental criticality data	good	N	
		Exponential Measurements on Light Water	H. J. Kouts, J. Chernick, I.					report, original,		
70	BNL-2094	Moderated 1 Per Cent U-235 Lattices	Kaplan	11/28/1952	BNL	Unclassified	experimental criticality data	good	N	
								ľ		
		Buckling of Light-Water Moderated Lattices of	H. Kouts, G. Price, K.					report, original,		
71	BNL-2184	.387" Diameter, 1.027% Enriched Uranium Rods	Downes, R. Sher, V. Walsh	02/07/1955	BNL	Unclassified	experimental criticality data	good	N	
	DIVE 2104	isor blameter, 1.027/0 Ennerted Gramam Roas	Downes, It. Sher, V. Walsh	02/07/1333	DIVE	Officiassifica	experimental criticality data	memo, original-		Measurements of migration areas for
		Migration Areas of Fission Neutrons in Uranium-	H. Kouts, G. Price, K.			Confidential,		issue copy,		LEU rod lattices are compared to various
72	DNI 2110	1 -		12/15/1054	BNL		l over a vim a et al aviti a ality data	1	N	· ·
	BNL-2119	Water Lattices	Downes, R. Sher, V. Walsh	12/15/1954	DINL	ueciassileu 03/24/1960	experimental criticality data	good	N	diffusion theory models.
								memo, original-		
		Thermal Utilization, 0.387" Diameter, 1.15%	H. Kouts, G. Price, V.			Confidential,		issue copy,		Measurements of thermal utilization for
73	BNL-2754	Enriched Uranium Rods in Light Water	Walsh	04/03/1956	BNL	declassifed 12/20/57	experimental criticality data	good	N	LEU rod lattices.
								memo, original-		
		Thermal Utilization, 1.3% Enriched, 0.600"				Confidential,		issue copy,		Measurements of thermal utilization for
74	BNL-2840	Diameter Uranium Rods in Light Water	Herbert J. Kouts	07/10/1956	BNL	declassifed 12/20/57	experimental criticality data	good	N	LEU rod lattices.
		Thermal Utilizations of .600" Diameter, 1%	H. Kouts, G. Price, V.			Confidential,		report, original,		Measurements of thermal utilization for
75	BNL-2849	Uranium Rod Lattices	Walsh	05/24/1956	BNL	declassifed 12/20/57	experimental criticality data	good	N	LEU rod lattices.
	<del> </del>	Critical Assemblies of Light Water Moderated,			<del>-</del>		,		· · · · · · · · · · · · · · · · · · ·	Experiment design, plan, accident
		Slightly Enriched Uranium Rod Lattices at						report, original,		analysis, and safety requirements for
76	BNL-3145	Brookhaven Hazards Report	Herbert Kouts	02/28/1956	BNL	Unclassified	experiment plan/design	good	N	LEU rod lattice experiments
	DI4F-2142		TICIDETE ROUES	02/20/1730	DIAL	Jiiciassineu	experiment high/nesign	1500u	111	LEG TOU lattice experiments
		Critical Assemblies of Light Water Moderated,								Europeded company of DNU 24.45 (
	2445 (6	Slightly Enriched Uranium Rod Lattices at		00/44/4555				report, original,		Expanded scope of BNL-3145 to cover
77	BNL-3145 (Suppl.)	Brookhaven Supplement to Hazards Report	Herbert Kouts	09/11/1956	BNL	Unclassified	experiment plan/design	good	N	smaller-diameter fuel rods.
								report,		Comprehensive experiment analysis for
		Safety Analysis of the Brookhaven National	A. Court, K. W. Downes, H.			Distribution of this		facsimile copy,		Brookhaven thermal and fast critical
78	BNL 11636	Laboratory Critical Assembly Facility	J. C. Kouts	08/01/1967	BNL	document is unlimited	experiment plan/design	good	Υ	experiments.
										Organized presentation of a large body
										of experimentally-determined assembly
		Uranium-Water Lattice Compilation Part I, BNL						report, original,		parameters, includes several BNL report
79	BNL 50035 (T-449)	Exponential Assemblies	Glenn A. Price	12/30/1966	BNL	[None]	experimental criticality data	good	Υ	as appendices.
	DIVE 30033 (1=443)		Sicili A. Frice	12/30/1300	DIAL	[INOIIE]	experimental criticality udid	- <del></del>	1	as appendices.
		Multilevel Analysis of the U <sup>235</sup> Total and Fission						report, original,		
80	BNL 50045 (T-455)	Cross Sections in the Energy Range Below 37 eV	ID. B. Alder, F. T. Adler	03/1967	BNL	[None]	nuclear measurement/data	good	N	

Evaluation of the Neutron Cross Section of Manganese for the Neutron Cross Section of Manganese for the ENDF/B Library S. Pearlstein O6/1967 BNL [None] nuclear measureme O6/1967 BNL [None] nuclear measureme O6/1967 BNL [None] nuclear measureme O7/1968 BNL [None] nuclear	report, original, good report, original, good ality data good report, original, good	N N N	First assembly to be taken critical in the Pulsed Fast Reactor facility - reports critical configuration and various measured reactor physics values.  Exponential measurements of buckling for Pu-Al rods in water  Monte Carlo code developed for infinite
Optical Model Analysis of the Elastic Scattering of Neutrons by the Lead Isotopes and Bismuth at 0.5, 1.0 and 2.5 MeV S. O. Moore 10/1968 BNL [None] nuclear measurements and Assembly Description U.5, 1.0 and 2.5 MeV S. O. Moore 10/1968 BNL [None] nuclear measurements and Assembly Description U.5, 1.0 and 2.5 MeV S. O. Moore 10/1968 BNL [None] nuclear measurements and Assembly Description U.5, 1.0 and 2.5 MeV S. O. Moore 10/1968 BNL [None] nuclear measurements and Assembly Description U.5, 1.0 and 2.5 MeV S. O. Moore 10/1968 BNL [None] nuclear measurements and Assembly Description U.5, 1.0 and 2.5 MeV Septimental Critical U.5, 1.0 and 2.5 MeV Septimenta	report, original, good	N N	Pulsed Fast Reactor facility - reports critical configuration and various measured reactor physics values. Exponential measurements of buckling for Pu-Al rods in water Monte Carlo code developed for infinite
of Neutrons by the Lead Isotopes and Bismuth at 0.5, 1.0 and 2.5 MeV  S. O. Moore  10/1968  BNL  [None]  nuclear measurements and 2.5 MeV  CEPFR-1A Measurements and Assembly  J. P. Phelps, A. J. Court, K.  BNL-RP-6 (F)  Description  W. Downes  09/22/1969  BNL  Ilimited distribution  experimental critical official Use Only, limited distribution  experimental critical official Use Only, limited distribution  experimental critical official Use Only, limited distribution  A Monte Carlo Study of Homogeneous  Plutonium and Uranium Mixtures  L. L. Carter, C. R. Richey  12/1964  PNNL  Unrestricted  Computational method is tribution  (2)	report, original, good ality data good report, original, good report, original, good thods/data report, original,	N N	Pulsed Fast Reactor facility - reports critical configuration and various measured reactor physics values. Exponential measurements of buckling for Pu-Al rods in water Monte Carlo code developed for infinite
82 BNL 50151 (T-520) 0.5, 1.0 and 2.5 MeV S. O. Moore 10/1968 BNL [None] nuclear measurements and Assembly J. P. Phelps, A. J. Court, K. Description W. Downes 09/22/1969 BNL limited distribution experimental critical official Use Only, limited dist	report, original, good ality data good report, original, good report, original, good thods/data report, original,	N N	Pulsed Fast Reactor facility - reports critical configuration and various measured reactor physics values. Exponential measurements of buckling for Pu-Al rods in water Monte Carlo code developed for infinite
CEPFR-1A Measurements and Assembly  Description  BNL  Description  Des	report, original, good report, original, good state good	N N	Pulsed Fast Reactor facility - reports critical configuration and various measured reactor physics values. Exponential measurements of buckling for Pu-Al rods in water Monte Carlo code developed for infinite
BNL-RP-6 (F) Description W. Downes 09/22/1969 BNL limited distribution experimental critica Official Use Only, limited distribution experimental critica Official Use Only, limited distribution experimental critica Official Use Only, limited distribution experimental critica  A Monte Carlo Study of Homogeneous BNL A Monte Carlo Study of Homogeneous Plutonium and Uranium Mixtures L. L. Carter, C. R. Richey 12/1964 PNNL Directricted Unrestricted Unrestricted Unrestricted	ality data good report, original, good thods/data report, original,	N	Pulsed Fast Reactor facility - reports critical configuration and various measured reactor physics values. Exponential measurements of buckling for Pu-Al rods in water Monte Carlo code developed for infinite
BNL-RP-6 (F) Description W. Downes 09/22/1969 BNL limited distribution experimental critica Official Use Only, limited distribution experimental critica Official Use Only, limited distribution experimental critica Official Use Only, limited distribution experimental critica  A Monte Carlo Study of Homogeneous BNL A Monte Carlo Study of Homogeneous Plutonium and Uranium Mixtures L. L. Carter, C. R. Richey 12/1964 PNNL Directricted Unrestricted Unrestricted Unrestricted	ality data good report, original, good thods/data report, original,	N	critical configuration and various measured reactor physics values. Exponential measurements of buckling for Pu-Al rods in water Monte Carlo code developed for infinite
BNL-RP-6 (F) Description W. Downes 09/22/1969 BNL limited distribution experimental critica Official Use Only, limited distribution experimental critica Official Use Only, limited distribution experimental critica Official Use Only, limited distribution experimental critica  A Monte Carlo Study of Homogeneous BNL A Monte Carlo Study of Homogeneous Plutonium and Uranium Mixtures L. L. Carter, C. R. Richey 12/1964 PNNL Directricted Unrestricted Unrestricted Unrestricted	ality data good report, original, good thods/data report, original,	N	measured reactor physics values.  Exponential measurements of buckling for Pu-Al rods in water  Monte Carlo code developed for infinite
84 BNL-RP-8 (F) Pu-Al-H <sub>2</sub> O Buckling Measurements G. A. Price, H. H. Windsor 10/01/1969 BNL dimited distribution experimental critical A Monte Carlo Study of Homogeneous Plutonium and Uranium Mixtures L. L. Carter, C. R. Richey 12/1964 PNNL distribution (2)  Subcritical Measurements with Storage	report, original, good  thods/data report, original,	N	Exponential measurements of buckling for Pu-Al rods in water Monte Carlo code developed for infinite
84 BNL-RP-8 (F) Pu-Al-H <sub>2</sub> O Buckling Measurements G. A. Price, H. H. Windsor 10/01/1969 BNL limited distribution experimental critical A Monte Carlo Study of Homogeneous Plutonium and Uranium Mixtures L. L. Carter, C. R. Richey 12/1964 PNNL distribution (2)  Subcritical Measurements with Storage	ality data good thods/data report, original,	N	for Pu-Al rods in water Monte Carlo code developed for infinite
A Monte Carlo Study of Homogeneous  BS BNWL-3 Plutonium and Uranium Mixtures L. L. Carter, C. R. Richey 12/1964 PNNL distribution (2)  Subcritical Measurements with Storage	thods/data report, original,		Monte Carlo code developed for infinite
85 BNWL-3 Plutonium and Uranium Mixtures L. L. Carter, C. R. Richey 12/1964 PNNL distribution (2)  Subcritical Measurements with Storage Unrestricted			· ·
85 BNWL-3 Plutonium and Uranium Mixtures L. L. Carter, C. R. Richey 12/1964 PNNL distribution (2)  Subcritical Measurements with Storage Unrestricted			
85 BNWL-3 Plutonium and Uranium Mixtures L. L. Carter, C. R. Richey 12/1964 PNNL distribution (2)  Subcritical Measurements with Storage Unrestricted			systems was used to calculate k-infinity,
85 BNWL-3 Plutonium and Uranium Mixtures L. L. Carter, C. R. Richey 12/1964 PNNL distribution (2)  Subcritical Measurements with Storage Unrestricted			limiting critical concentration, etc., as
85 BNWL-3 Plutonium and Uranium Mixtures L. L. Carter, C. R. Richey 12/1964 PNNL distribution (2)  Subcritical Measurements with Storage Unrestricted			well as physics/diffusion theory
Subcritical Measurements with Storage Unrestricted	8000	N	parameters (e.g., neutron age)
<u> </u>			1/M measurements were performed.
<u> </u>			The number of containers was adequate
	report, original,		to support operational limits but not
86 BNWL-19 Containers of Plutonium Nitrate Solution R. C. Lloyd, E. D. Clayton 01/1965 PNNL distribution experimental critical	1		
	ality data good	N	benchmark data.
Function Magnitude and Neutron			The number of elements (in a water
Exponential Measurements and Neutron			lattice) was adequate to support
Multiplication Measurements with 1.25 wt% C. L. Brown, R. C. Lloyd, S. Unrestricted	report, original,		operational limits but not benchmark
87 BNWL-52 Enriched N-Reactor Fuel Elements in Light Water R. Bierman, E. D. Clayton 03/1965 PNNL distribution experimental critica		N	data.
Physics Research Quarterly Report October, Unrestricted	report, original,		
88 BNWL-222 November, December 1965 Staff 01/14/1966 PNNL distribution multiple topics, TOC	C scanned good	N	
			Includes summaries for in-progress
			critical experiments (PuO <sub>2</sub> -UO <sub>2</sub> rods in
			water, arrays of 233 U solution in poly
			bottles, Pu nitrate solution in slab
			geometry), and k-infinity measurements
Pagetar Physics Department Technical Activities	report, original,		(graphite-moderated heterogeneous
Reactor Physics Department Technical Activities Unrestricted	' ' - '		
89 BNWL-340 Quarterly Report July, August, September 1966 Staff 10/15/1966 PNNL distribution multiple topics, TOC	C scanned good	Y	assemblies of Pu-Al-ThO <sub>2</sub> ).
			Includes summaries for in-progress
			critical experiments ( Pu nitrate solution
			in slab geometry, PuO <sub>2</sub> polystrene
			compacts), and k-infinity measurements
			(graphite-moderated heterogeneous
Reactor Physics Department Technical Activities			assemblies of Pu-Al-ThO <sub>2</sub> ). Also, has a
Quarterly Report January, February, March Unrestricted	report, original,		description of the manufacturing process
90 BNWL-775 1968 Staff 07/1968 PNNL distribution multiple topics, TOC		Υ	of Al-clad, Pu-Al fuel plates/assemblies.
Suit Office United Unit	- 300icu   8000	<u> </u>	Includes summaries for in-progress
	report, original,		critical experiments ( Pu nitrate solution
91 BNWL-921 September 1968 Staff 12/1968 PNNL distribution multiple topics, TOC		Y	in slab geometry).
September 200 Sail 12/1906 Print distribution infutiple (b)tis, 100	c scarnica good	<del>                                     </del>	Includes summaries for in-progress
			critical experiments ( PuO <sub>2</sub> polystrene
			compacts and depleted uranium, in
Reactor Physics Quarterly Report January, Unrestricted	report, original,		alternating layers, Pu nitrate solution in
92 BNWL-1053 February, March 1969 Staff 05/1969 PNNL distribution multiple topics, TOC	C scanned good	Υ	slab geometry).
			Mass Physics) and 6 (Publications).
			Includes summaries for in-progress
			critical experiments (PuO <sub>2</sub> compacts plus
			PuO <sub>2</sub> polystrene compacts and
			heterogeneous neutron absorbers). Also
			includes an assessment of criticality
			safety impacts on design of large-scale
Reactor Physics Quarterly Report April, May, Unrestricted	report, copy,		plants manufacturing PuO <sub>2</sub> -UO <sub>2</sub> fuel
93 BNWL-1150 June 1969 Staff 08/1969 PNNL distribution multiple topics	fair	Y	(LMFBR).

							Ţ	,		
										Copy includes only Section 5 (Critical
										Mass Physics) and 6 (Publications).
										Section 5 content is primarily results of
		Reactor Physics Quarterly Report July, August,				Unrestricted		report, copy,		code calculations (with comparisons to
94	BNWL-1240	September 1969	Staff	11/1969	PNNL	distribution	multiple topics	fair	N	critical experiment data).
										Includes summaries for in-progress
										critical experiments (PuO <sub>2</sub> -UO <sub>2</sub> rods in
										water. Also, computed criticality data
		Reactor Physics Quarterly Report October,				Unrestricted		report, original,		for higher actinides (e.g., ANS-8.15
95	BNWL-1304	November, December 1969	Staff	02/1970	PNNL	distribution	multiple topics, TOC scanned	good	Υ	actinides).
	DIVVE 1304	November, December 1909	Stair	02/13/0	TIME	distribution	matapic topics, roe scanned	good	· · · · · · · · · · · · · · · · · · ·	Includes summaries for in-progress
		Baratas Blassica Ossartaski Barast Assil Mari								critical experiments (PuO <sub>2</sub> -UO <sub>2</sub> rods in
0.5	2004 2	Reactor Physics Quarterly Report April, May,	C. II	00/4070	2010	fa. 1		report, original,	.,	
96	BNWL-1381-2	June 1970	Staff	08/1970	PNNL	[None]	multiple topics, TOC scanned	good	Υ	water, Pu-Al rods in water).
			D. H. Thomsen, T. M.				computational method/data	report, original,		
97	BNWL-1433	BMC-1: The Battelle Monte Carlo Code	Traver	06/1970	PNNL	[None]	(2)	good	N	
			C. L. Bennett, W. L.				computational method/data	report, original,		
98	BNWL-1434	BRT-1: Battelle-Revised-THERMOS	Purncell	06/1970	PNNL	[None]	(2)	good	N	
		Criticality Safety Analysis Model 60 Shipping						report, original,		
99	BNWL-1512	Package for FFTF Driver Fuel Pins	C. L. Brown	11/1970	PNNL	[None]	transport safety analysis	good	N	
		Technical Activities Quarterly Report AEC								
		Reactor Development and Technology Programs						report, original,		
100	BNWL-1522-3	April, May, June 1971	Staff	10/1971	PNNL	[None]	multiple topics, TOC scanned	good	N	
				,						miciades summaries for in-progress
										critical experiments (Pu nitrate solution,
		Technical Activities Quarterly Report AEC								Pu consists of 43% <sup>240</sup> Pu and 10.9%
		Reactor Development and Technology Programs						report, original,		
101	BNWL-1522-4	July, August, September 1971	Staff	12/1971	PNNL	[None]	multiple topics, TOC scanned	good	Y	<sup>241</sup> Pu).
		Technical Activities Quarterly Report AEC								Includes data/evaluation for Pu
		Reactor Development and Technology Programs								polystyrene compact critical experiments
		October, November, December 1971 January,						report, original,		and for high-burnup Pu solution
102	BNWL-1522-5	February, March 1972	Staff	05/1972	PNNL	[None]	multiple topics, TOC scanned	good	Υ	experiments (43% <sup>240</sup> Pu and 10.9% <sup>241</sup> Pu)
		DRAFT A Computer Code for the Calculation of				()	computational method/data	report, original,		
103	BNWL-1607	Fission Product Activity Ratios	D. R. Oden, G. D. Seybold	06/1971	PNNL	[None]	(3)	good	N	
	5.1112 2007	i ission i rodder receively racios	D. I Guerry G. D. Seybolu	00/13/1		[Hone]	(5)	Bood		Includes a section discussing the
										influence of experiment and data
		Uncertainties in the Analysis of Plutonium	R. C. Liikala, V. O. Uotinen,					report, original,		uncertainties on the ability to calculate
104	DNIMIL 1CEC			05/1072	DAINI	[None]	oungrimental criticality data	1	N	•
104	BNWL-1656	Fueled Light Water Moderated Assemblies	U. P. Jenquin	05/1973	PNNL	[None]	experimental criticality data	good	N	keff for critical experiments.  Irradiation test of a (miniature) rocket
										i i
										motor with solid propellant. A fast
						Distribution limited to				reactor pulse is performed as the rocket
		Nuclear Engineering Problems of a Burning			Aberdeen Proving	U. S. Government		report, original,		motor is firing, with the motor and
105	BRL MR 2217	Rocket Motor Neutron Irradiation	A. H. Kazi, J. L. Watson	08/1972	Ground	agencies	operational/test/material data	good	N	reactor in very close proximity.
					Commissariat Á					
					l'Énergie Atomique,					
					Centre D'Études					In French. Use of photographic film to
		Mesure Des Densités de Neutrons Par			Nucléaries (Saclay,			report, original,		measure neutron flux distribution for
106	CEA No. 204	Autoradiographie De Détecteurs	A. Ertaud, E. Zaleski	1953	France)	[None]	nuclear measurement/data	good	N	experiments.
					<u> </u>	1	1	report, copy		
			Le Groupe de Travail nº 4		Commissariat Á			from		
		Surete - Criticite Recommandations Concernant	de la Sous-Commission		l'Énergie Atomique		facility, process or storage	microfiche,		In French. General guidelines for storage
107	CEA-N-1291	Les Stockages De Matiere Fissile	des Masses Critiques	06/1970	(Paris, France)	[None]	analysis	poor	N	of fissile material.
		sconages se madere rissine	masses oritiques	33,1370	(, a., 5, 1 tallee)	[5]		- 30.	.,,	
					Metallurgical					
					Laboratory					
					,	Cocrot doclarsification				
					(Manhattan Project	Secret, declassification				Markarial assessment 1 1 100
					section at the	indicated but no date		report, original,		Material preparation, solubility,
108	CC-2092	Properties of Uranyl Fluoride	G. R. Dean	09/11/1944	University of Chicago)	provided	operational/test/material data	good	N	conductivity, chemical properties
					Combustion					
		Studies on Low Enriched Cores Containing	J. S. Crudele, C. O.		Engineering, Inc.,			report, copy		
	1	Simple Nuclear Superheat Elements Part I -	Dechand, P. G. Klann, W.		Nuclear Division,			from microcard,		

			T					T		
110	CP-2842	Water Lattice Experiments	A. M. Weinberg, Haydn Jones	10/26/1945	ORNL	Secret, declassified 05/23/1955	experimental criticality data	report, original, good	N	Attempts to determine if criticality could be attained with normal enrichment uranium (uranium as metal rods in lattice). Exponential measurements done in a water tank located on top of the Oak Ridge Graphite Reactor.
					Metallurgical					
					Laboratory					Exponential measurements performed in
					(Manhattan Project					a tank located on top of reactor CP-2.
		Exponential Measurements with D <sub>2</sub> O and			section at the	Secret, declassified		report, original,		Measurements were done on near-pure
111	CP-3364	Solutions of UO <sub>2</sub> F <sub>2</sub>	A. Wattenburg	02/02/1946	University of Chicago)	01/18/1955	experimental criticality data	good	N	D <sub>2</sub> O and uranyl fluoride in D <sub>2</sub> O solution.
										for VERA assemblies using either
										enriched uranium (effective enrichment
										of 32% <sup>235</sup> U) or Pu (5% <sup>240</sup> Pu). Some U
		Weapons Group Nuclear Research Division								assemblies contained some polythene. Also summaries for Pu-polythene
		Progress Report 1st. January 1963 - 30th. June			Aldermaston A.E.R.E.			report, original,		
112	CNR/PR/1	1963	Staff	02/1964	(U.K.A.E.A.)	Distribution	multiple topics	good	N	assemblies at H: <sup>239</sup> Pu of 0, 3.0, and 6.1. Includes one additional critical
										experiment summary for a VERA Pu
		Weapons Group Nuclear Research Division								assembly, of the experiment series
		Progress Report 1st. July 1963 - 31st. December			Aldermaston A.E.R.E.	Unclassified, Limited		report, original,		described in CNR/PR/1. Also, notes on
113	CNR/PR/2	1963	Staff	04/1964	(U.K.A.E.A.)	Distribution	multiple topics	good	N	evaluation of Pu-polythene assemblies.
		Weapons Group Nuclear Research Division								
	CND (DD (D	Progress Report 1st. January 1964 - 30th. June	c. "	40/4054	Aldermaston A.E.R.E.		le to the second	report, original,		
114	CNR/PR/3	1964   Weapons Group   Nuclear Research Division	Staff	10/1964	(U.K.A.E.A.)	Distribution	multiple topics	good	N	
		Progress Report 1st. July 1964 - 31st. December			Aldermaston A.E.R.E.	Unclassified, Limited		report, original,		
115	CNR/PR/4	1964	Staff	03/1965	(U.K.A.E.A.)	Distribution	multiple topics	good	N	
		Weapons Group Nuclear Research Division						ľ		
		Progress Report 1st. January 1965 - 30th. June			Aldermaston A.E.R.E.	Unclassified, Limited		report, original,		
116	CNR/PR/5	1965	Staff	09/1965	(U.K.A.E.A.)	Distribution	multiple topics	good	N	
		Weapons Group Nuclear Research Division			Alderse A.F.D.F.	Harland Bark				
117	CNR/PR/6	Progress Report 1st. July 1965 - 31st. December 1965	Staff	03/1966	Aldermaston A.E.R.E. (U.K.A.E.A.)	Unclassified, Limited Distribution	multiple topics	report, original, good	N	
117	CIVITY PINY O	Weapons Group Nuclear Research Division	Stall	03/1900	(U.K.A.E.A.)	Distribution	multiple topics	good	IN	
		Progress Report 1st. January 1966 - 30th. June			Aldermaston A.E.R.E.	Unclassified, Limited		report, original,		
118	CNR/PR/7	1966	Staff	10/1966	(U.K.A.E.A.)	Distribution	multiple topics	good	N	
		Weapons Group Nuclear Research Division								
	CND (DD (O	Progress Report 1st. January 1967 - 30th. June	c. "	00/4057	Aldermaston A.E.R.E.	,	10.1	report, original,		
119	CNR/PR/9	1967 Weapons Group Nuclear Research Division	Staff	09/1967	(U.K.A.E.A.)	Distribution	multiple topics	good	N	
		Progress Report 1st. July 1967 - 31st. December			Aldermaston A.E.R.E.	Unclassified, Limited		report, original,		
120	CNR/PR/10	1967	Staff	04/1968	(U.K.A.E.A.)	Distribution	multiple topics	good	N	
					ORNL (Computing					
					Technology Center),					
121	CTC F	KENO - A Multigroup Monte Carlo Criticality	G. E. Whitesides, N. F.	00/40/4050	Union Carbide	[Name]	computational method/data	report, original,		
121	CTC-5	Program	Cross	09/10/1969	Corporation	[None]	(2)	good	N	Experiments performed at ORNL Bldg
122	DC-51-9-11	Report on Critical Experiments for a Water Moderator (CA-2)	J. A. Hunter (*)	1951	General Electric Company	Secret, declassified 12/04/1961	experimental criticality data	report, original, good	Y	29213. Thin plates of U(93) metal interspersed with graphite and plexiglas blocks to simulate aircraft reactor concepts. The author's name does not appear in the report (see ORNL-1175 p. 7, footnote)
122	DEC D+ 35 (2)	Critical Mass Data for Low-Enriched Uranium	5 D Charles "	4050	U.K.A.E.A. (Risley,	@ Off:-:-1:: 0 :	computational method/data	report, original,		
123	DEG Report 25 (R)	Systems	F. R. Charlesworth	1959	Lancashire)	©, Official Use Only	(1), equipment/process design	gooa	N	

		1				Not to be	T	1 1		
						communicated to				
						any person not				Discusses need for Pu integral data to
		The Effect of Isotopic Content on the Criticality			U.K.A.E.A. (Risley,	authorized to receive	computational method/data	report, original,		support credit for <sup>240</sup> Pu content, for
124	DEG Report 82 (R)	of Plutonium-Water Systems	D. E. J. Thornton	1960	Lancashire)	it.	(1)	fair	N	design/operation of processing facilities.
	DEG REPORT OF (IV)	The state of the s	Di Elsi Monton		zancasını cy	Not to be	(-)	10		acsign, operation of processing radinates.
						communicated to				
		Criticality Data for Homogeneous Uranium-				any person not				Uses diffusion theory, buckling
		Water Mixtures of Enrichments in the Range 5 to			U.K.A.E.A. (Risley,	authorized to receive	computational method/data	report, copy,		conversions, etc. to calculate simple-
125	DEG Report 153 (R)	93%	M. A. Perks	1960	Lancashire)	it.	(1), equipment/process design	fair	N	geometry limits for design applications.
						Not to be				
						communicated to				Uses computed results from U.K.A.E.A.
		Criticality Guide to the Choice of Safe Vessel				any person not				report IG Memorandum 462 to provide
		Dimensions for the Processing and Storage of			U.K.A.E.A. (Risley,	authorized to receive	computational method/data	report, original,		guidance for designs of dissolvers and
126	D.E.G. Memorandum 329 (R)	U <sup>235</sup> and Pu <sup>239</sup>	F. R. Charlesworth	11/23/1959	Lancashire)	it.	(1), equipment/process design	good	N	solution storage vessels.
										Uses U. S. and British LEU experiments,
										applies diffusion/buckling methods to
		1 -	M. A. Perks, D. E. J.		U.K.A.E.A. (Risley,		computational method/data	report, original,		calculate simple-geometry limits for
127	D.E.G. Memorandum 417 (R)	Enrichment Below 2.5%	Thornton	02/1960	Lancashire)	Official Use Only	(1), equipment/process design	good	N	chemical plant designs.
						Unclassified, Not to be				
						communicated to				
		Criticality of 200/ Family ad Handison Calettana in	L C Contab. A M Doubles I		LLIKAEA (Distan	any person not				Experiments are similar to those of
128	DEC Managed (C2/D)	1	J. C. Smith, A. V. Parker, J.	03/1960	U.K.A.E.A. (Risley,	authorized to receive		report, copy,	Υ	IHESCBE report IEU-SOL-THERM-002, but
128	DEG Memorandum 663(D)	Cylindrical Geometry - Interim Report	G. Walford, C. White	03/1960	Lancashire)	IT.	experimental criticality data	fair	Y	in cylindrical geometry.  Bucking relationships were
						Not to be				derived/applied to three sets of LEU rod
						communicated to				experiments of differing enrichments,
						any person not				then used to predict critical parameters
		Lattice Calculations for Slightly Enriched			U.K.A.E.A. (Risley,	authorized to receive	computational method/data	report, copy,		within the enrichment range and lattice
129	DEG Memorandum 812 (R)	Uranium Oxide - Light Water Assemblies	S. Barnett	04/1960	Lancashire)	it.	(1), equipment/process design	fair	N	parameters.
		0			,		////	-		Experimental measurements for D <sub>2</sub> O-
		Neutron Age in Mixtures of Light and Heavy						report, original,		H <sub>2</sub> O mixtures ranging from 92 to 100%
130	DP-163	Water	James W. Wade	06/1956		Unclassified	nuclear measurement/data	good	N	D <sub>2</sub> O.
	DI 103	water	James W. Wade			Officiassifica	- Indical measurement data	good		Empirical relationships derived and fitted
							computational method/data	report, original,		to experiment data to provide guidance
131	DP-132	Interaction of Subcritical Components	H. K. Clark	11/1958	SRNL	[None]	(1), equipment/process design		N	for criticality designs.
	DI 132	interaction of Suberitical Components	TI. K. Clark	11/1550	SINVE	[IVOIIC]	(1), equipment, process design	good		Measurements of lattice physics
										parameters for a lattice of 3-inch dia
		Nuclear Parameters of Massive Uranium Rods in						report, original,		U(3) metal rods in D <sub>2</sub> O, with 18-in
132	DP-644	D <sub>2</sub> O	F. E. Kinard	11/1961	SRNL	[None]	experimental criticality data	good	N	triangular pitch
		-2-				,				Measurements of cadmium ratios and
										self-shielding for foils of gold, indium,
		A Study of Thermal and Resonance Neutron Flux						report, original,		tugsten, eropium, manganese, lutecium,
133	DP-608	Detectors	G. M. Jacks	08/1961	SRNL	[None]	nuclear measurement/data	good	N	and dysprosium.
		Comparison of a Simple Theoretical Treatment		· · · · · · · · · · · · · · · · · · ·						
		of Critical Arrays of Fissionable Units with					computational method/data	report, original,		The correlation method uses diffusion
134	DP-868	Experiment	H. K. Clark	01/1964	SRNL	[None]	(1)	good	N	theory, solid angle, and albedoes.
										The FORTRAN code implements a
		Interaction of Fissile Units A Computer Code -					computational method/data	report, original,		computational method similar to that of
135	DP-1031	INTERACT	H. K. Clark	06/1966	SRNL	[None]	(1)	good	N	DP-868.
										Rods were fabricated of PuO <sub>2</sub> mixed with
										depleted UO <sub>2</sub> to simulate heavy metal
										oxide content of irradiation natural
			N. P. Baumann, J. L.							uranium fuel. Measurements included
			Crandall, R. L. Olson, G. F.							buckling, temperature effects on
		Lattice Experiments with Simulated Burned-Up	O'Neill, D. J. Pellarin, V. D.				experimental criticality data,	report, original,		buckling, thermal neutron distributions,
136	DP-1122	Fuel for D <sub>2</sub> O Power Reactors	Vandervelde	02/1968	SRNL	[None]	nuclear measurement/data	good	N	conversion ratios, etc.

		<b>,</b>					-p	,		
										Measurements involving only rods
										containing thorium were far subcritical
										and limited to lattice physics
										parameters. The critical experiments
										involved substitution measurements
										where only a fraction of the fuel rods
		Zero Power Experiments with <sup>235</sup> U-Enriched	D. J. Pellarin, N. P.							contained thorium. "HWOCR" was an
		Thoria and Thorium Metal Lattices for the	Baumann, J. L. Crandall, G.				experimental criticality data,	report, original,		ANL acronym for Heavy Water Organic-
137	DP-1125	HWOCR	F. O'Neill, R. M. Satterfield	11/1967	SRNL	[None]	nuclear measurement/data	good	N	Cooled Reactor."
137	5. 1125		T. O Mem, III IVII Succernetu	11,130,		[itolic]	nacical measurement, auto	Bood		Development of a generalized
								report, original,		computational model (and computer
138	DP-1168	Quantitative Analysis of Reactor Safety	L. M. Arnett	11/1968	SRNL	[None]	oversiment safety analysis	good	N	code) to perform probabilistic analysis.
130	DF-1106	Qualititative Analysis of Reactor Safety	L. IVI. AITIELL	11/1506	JUINT	[NOTIE]	experiment safety analysis	good	IN	Development of computational method
										(and computer code) to model neutron
										interaction in large, heterogeneous,
	1	Calculation of Relative Reaction Rates in					computational method/data	report, original,		thermal nuclear reactors (e.g., the
139	DP-1238	Interacting Cylindrical Cells	H. K. Clark	09/1970	SRNL	[None]	(2)	good	N	Savannah River Plant reactors).
										The Nuclear Test Gauge was used to
							1			confirm reactivity of inserted fuel items
										or assemblies; it was intended to remain
										subcritical in all operating modes. The
										consequences of a potential criticality
		Termination of Nuclear Transients in the Nuclear						report, original,		accident (due to insertion of off-
140	DP-1315	Test Gauge	J. P. Church, W. S. Durant	01/1973	SRNL	[None]	experiment safety analysis	good	N	specification items) are evaluated.
			·							Paper was presented at a nuclear safety
										conference held at Savannah River
							facility/process/storage	paper, original,		Laboratory in March 1959, copy
141	DPST-59-445	Extrapolating Data and Margins of Safety	H. K. Clark	03/1959	SRNL	[None]	analysis	good	N	transmitted by memo to Dixon Callihan.
171	DI 31 33 443	Extrapolating Data and Margins of Sarcty	TI. K. Clark	03/1333	JIME	[None]	unutysis	good		This document is LA-1159. Wording
										referring to "hydride weapons" and the
										document distribution list have been
										removed (cut out). The current LANL
										1
										website version of LA-1159 is marked
								report, original		"Approved for Public Release." This
								but with		appears to have been a declassified
			H. C. Paxton, J. D. Orndoff,			Secret, declassified		excerpted		version provided to Callihan by Savannah
142	D-RL-902	Oralloy Hydride Critical Assemblies	G. A. Linenberger	05/22/1950	SRNL	05/01/1973	experimental criticality data	sections, good	Υ	River Laboratory.
										This document is LA-1251. Wording
										referring to tamper "compression" and
										the document distribution list have been
							1			removed (cut out). LA-1251 was not
								report, original		found on LANL external website. This
								but with		appears to have been a declassified
		Critical Masses of Oralloy at Reduced	J. D. Orndoff, H. C. Paxton,			Secret, declassified	1	excerpted		version provided to Callihan by Savannah
143	D-RL-903	4	G. E. Hansen	05/01/1951	LANL	05/01/1973	experimental criticality data	sections, good	Υ	River Laboratory.
				, , , , , , , , , , , ,		. ,		1, 5		This document is LA-1356(Del).
										Distributed as Reference 135 of the LA-
										10680-MS reference set. Some words
										within the document and the document
										distribution list have been removed (cut
										1
								roport origin-1		out). LA-1356(Del.) was not found on
								report, original		the LANL external website. This appears
		Precision Critical-Mass Determinations for					1	but with		to have been a declassified version
		Oralloy and Plutonium in Spherical Tuballoy	l			Secret, declassified	i e	excerpted		provided to Callihan by Savannah River
144	D-RL-904		G. E. Hansen, D. P. Wood	01/01/1951	LANL	05/01/1973	experimental criticality data	sections, good	Υ	Laboratory.

									ı	Plans for short-term use of a bare
									1	assembly of HEU metal from Oak Ridge,
									1	to be pulsed by accelerator-produced
						Reproduction is			1	neutrons. Purpose is to design a similar
					John Jay Hopkins,	permitted for any			ı	metal assembly for the GA accelerator to
						1			ı	•
		Hazards Summary Report for the Metal		/ /	Laboratory, General			report, original,		testing of transient radiation effects on
145	GA-4488	Assembly	J. R. Brown, J. L. Russell	08/15/1963	Atomic, San Jose CA	States Government.	experiment plan/design	good	N	electronics.
			J. L. Russell, Jr., J. R.						1	
			Brown, D. F. Herring, H. A.						1	Similar to GA-4488 except the primary
		Feasibility Study of the Accelerator Pulsed Fast	Kazi, J. U. Koppel, H. D.		John Jay Hopkins,				ı	
		Assembly Technical Summary Report October	Smith, W. P. Wallace, P. B.		Laboratory, General			report, original,	ı	focus is on the final GA assembly (which
146	GA-4873	1, 1962 through September 30, 1963	Wilson	12/31/1963	Atomic, San Jose CA	Official Use Only	experiment plan/design	good	N	would likely be a 233U metal assembly).
										Uses diffusion theory, buckling, four-
									ı	factor formulas, and solid-angle method
									1	to provide computed results for bare
							computational method/data	report, original,	1	solution cylinder/cylinder array criticalit
147	GAT-189	Critical Geometries for Bare Cylinders	J. A. Pond	07/20/1956	Portsmouth GDP	[None]	I .	1	N	requirements.
147	GA1-169	A Re-Study of the Uranium Concentration or	J. A. POHU	07/20/1956	POLISIIIOULII GDP	[None]	(1)	good	IN .	
		1							ı	Correlations of U density and H:U atom
		Density Values Over the Full Range of						memo, original,	I	ratio for moderated uranyl fluoride
148	GAT-532-67-66	Hydrogenous Moderations, for UF <sub>6</sub> and UO <sub>2</sub> F <sub>2</sub>	J. L. Feuerbacher	05/24/1967	Portsmouth GDP	[None]	operational/test/material data	good	N	mixtures.
										Proposes increases in enrichment limits
		A New Study of Nuclear Hazards of Process Gas					computational method/data	memo, original,	ı	for gas coolers in the Portsmouth
149	GAT-DM-639, Add. 1	Coolers, Add. 1	J. L. Feuerbacher	09/17/1959	Portsmouth GDP	[None]	(1), equipment/process design	good	N	diffusion plant.
							( ), - ( - ),			Proposes increased geometric and mass
									ı	limits in the noted enrichment range.
									ı	Folder includes a carbon-copy of a
									ı	"Memo to File" recording phone
									ı	
									ı	conversations wherein Oak Ridge
									I	recipients of the memo expressed
		Cofe Connectains and Many at Assess Balance Since							ı	concern to Portsmouth management re
		Safe Geometries and Mass at Assays Below Five					computational method/data	memo, original,	ı	technical content and mass limit
150	GAT-DM-769	Per Cent U <sup>235</sup>	J. L. Feuerbacher	05/18/1959	Portsmouth GDP	[None]	(1), equipment/process design	good	N	increases of GAT-DM-769.
		Increased Safe Pipe Diameter and Annulus								Uses ANISN calculations to justify
		Thickness Based on the Use of Neutron Absorber					computational method/data	memo, original,	ı	increased dimensional limits for process
151	GAT-DM-1127	Shields or Alloys	J. L. Feuerbacher	11/27/1968	Portsmouth GDP	[None]	(2), equipment/process design	good	N	vessels.
							, , , , , , , , , , , , , , , , , , , ,			
									I	Extrapolation length values are
		Reflector Savings for Homogeneous Aqueous					computational method/data	memo, original,	ı	calculated for various reflector and other
152	CAT DD 375		John A. Dond	01/12/1050	Doutsmouth CDD	[None]	1			
152	GAT-DR-275	Solutions of U-235	John A. Pond	01/13/1958	Portsmouth GDP	[None]	(1), equipment/process design	good	N	conditions, based on two-group theory.
									ı	Proposes higher safe/subcritical mass
		Estimated Minimum Critical Mass and Safe							1	limits for moderated uranyl fluoride at
		Parameters of Uranium Below 5% U-235 Assay in					computational method/data	memo, original,	l	2.0 and 1.2 % enrichment based on
153	GAT-T-551	Mixtures of Uranyl Fluoride and Water	J. L. Feuerbacher	09/11/1958	Portsmouth GDP	[None]	(1), equipment/process design	good	N	diffusion/buckling techniques.
		Rapid Calculation of Maximum Nuclear								Method is simple calculation of
		Reactivity of Homogeneous Uranyl Fluoride -					computational method/data	memo, original,	ı	geometric buckling, application of
154	GAT-T-670/Rev. 1	Water Mixtures at All U-235 Enrichments	J. L. Feuerbacher	10/15/1959	Portsmouth GDP	[None]	(1), equipment/process design	good	N	leakage and tabulated k-infinity terms.
	· ·					· · · · · · · · · · · · · · · · · · ·	7, 7, 7, 7, 7, 7, 7, 7, 7, 7, 7, 7, 7, 7	Ī		Basic use of thermal cross sections and
		Moderating Ratios and Poisoning Ratios of the					computational method/data	report, original,	ı	atom masses to tabulate the noted
155	GAT-T-673	Elements and Some Compounds	J. L. Feuerbacher	09/01/1959	Portsmouth GDP	[None]	1		N	ratios. No guidance on usage.
133	UA1-1-0/3	Elements and some compounds	J. L. Federbacher	09/01/1959	FOLISHIOUTH GDA	[INOTIE]	(1)	good	IN	
							1		1	Basic use of a six-factor formula, with
		Practical Methods for Calculating Reactivity of		1			computational method/data	report, original,	ı	terms read from graphs or calculated, to
156	GAT-T-692	Homogeneous Uranium Compounds	J. L. Feuerbacher	10/15/1959	Portsmouth GDP	[None]	(1)	good	N	perform keff estimates.
					Vallecitos Atomic					
		Neutron Thermalization Calculations for a			Laboratory, General				1	Assessment of methods/data used to
		Heterogeneous Lattice Containing Uranium and	P. Greebler, W. H. Harker,		Electric, Pleasanton		computational method/data	report, original,	ı	account for non-1/v absorption effects i
157	GEAP-3001	Plutonium Fuel in Water	J. M. Harriman	04/14/1958	CA	[None]	(1)	good	N	Pu-239.
		AEC Superheat Criticals - A Comparison of	G. T. Petersen, F. G.	2 ., 2 ., 2330		,	computational method/data	report, original,		
158	GEAP-3882	Experiment and Theory on Uniform Lattices	Warzek	01/1962		[None]	(1)	good	N	
130	OLAF -3002	Experiment and Theory of Official Lattices	VV G. LCR	01/1302	1	[140116]	\±/	500u		

			T	Τ	Health and Safety	T	T	1		
			H. Beck, G. Burke, J.		Laboratory, U.S.					
			DeCampo, C. Gogolak, F.		Atomic Energy					Measurements and analysis for normal-
		HASL Studies at the Oyster Creek Nuclear	Hajnal, W. Lowder, J.		Commission, New			report, original,		operations radionuclide releases from a
159	HASL TM 72-3	Electricity Generating Station	McLaughlin, P. Raft	07/1972	York, NY	[None]	nuclear measurement/data	good	N	BWR plant.
					Health and Safety					238 232
					Laboratory, U.S.					<sup>238</sup> U and <sup>232</sup> Th are significant contributor
		Absolute Intensities of Gamma Rays from the			Atomic Energy					to background radiation and thus
		1			Commission, New			report, original,		interfere with the ability to accurately
160	HASL-262	Decay of <sup>238</sup> U and <sup>232</sup> Th	Harold L. Beck	10/1972	York, NY	[None]	nuclear measurement/data	good	N	measure normal reactor emissions.
						Applied Technology:				
						" further distribution				
						of this document to				
						third parties				
			R. Gold, J. H. Roberts, L. S.			representing foreign				
			Kellogg, W. Y. Matsumoto,		Hanford Engineering	interests should be				
		Reaction Rate Measurements un the CFRP/PNC	W. N. McElroy, C. C.		Development	coordinated with the	experimental criticality data,	report, original,		Limited description of the critical
161	HEDL-TME 86-3	Criticality Experiments Program (Task 20)	Preston, R. L. Simons	05/1987	Laboratory	USDOE"	nuclear measurement/data	good	N	experiments is provided.
								8		Uses the code "GEM." Folder includes
										three pages labeled "Comparison with
		Health and Safety Branch, Technical Section	A. J. Roskell, D. W.							ORNL Results". It is not clear whether
		Criticality Results Bulletin No. 15, Some Critical	Tattersall, T. Murphy, E. R.				computational method/data	report, original,		these pages are part of the technical
162	UC/CD/100F/DD1F	1	Woodcock	10/26/1062	U.K.A.E.A.	Official Has Only	1		NI.	
162	HS/CR/1095/RB15	Arrays of Spherical Fissile Units in Air	VVUOULULK	10/26/1962	U.N.A.E.A.	Official Use Only	(1)	fair	N	bulletin/report.  Uses the code "GEM." Folder includes
		Health and Safety Branch, Theoretical and								three pages labeled "Comparison with
		Computing Section Results Bulletin No. 27,								ORNL Results". It is not clear whether
							computational method/data	report, original,		these pages are part of the technical
163	HS/CR/1095/RB27	Pu <sup>240</sup> Poisoning in Aqueous Plutonium Solutions	E. R. Woodcock	12/1964	U.K.A.E.A.	NOT FOR PUBLICATION	(1)	fair	N	bulletin/report.
					Hanford Works,					Outlines application of K-643
		Results of Critical Mass Studies for Hanford at K-			General Electric,	Secret, declassified		report, original,		experimental results for criticality
164	HW-19112	25	Paul F. Gast	10/11/1950	Richland, WA	01/30/1958	equipment/process design	fair	N	concerns at Hanford.
										Report describing the criticality accident
										of November 16, 1951, during critical
		A Study of the Radiation Burst in the Hanford				Secret, declassified		report, original,		experiments with a spherical tank of Pu
165	HW-24327	Homogeneous Reactor	B. R. Leonard, Jr.	05/02/1952	Hanford	12/27/1957	criticality accident	good	N	solution.
		Proposed Method for Treating Hydrogen				Confidential,		report, negative		
		Displacement Effects in Critical Mass				declassified	computational method/data	photostatic		
166	HW-24454	Measurements	P. F. Gast	05/19/1952	Hanford	05/31/1960	(1)	copy, fair	N	Uses diffusion theory methods.
				<u> </u>				1		HW-24514 was originally issued in 1952
										as a classified document. The copy in
			F. E. Kruesi, J. O. Erkman,					report, original,		Callihan's files is the declassified ("DEL")
167	HW-24514 DEL	Critical Mass Studies of Plutonium Solutions	D. D. Lanning	05/19/1952	Hanford	Unclassified	experimental criticality data	good	Υ	reissue of 02/15/1960.
107	2.52.522	and the second of the content solutions		33, 13, 1332				8500		. 5.5340 0. 02/13/1500.
			D. J. Donahue, D. D.			Secret, declassification		report, negative		
		Summary Report of the Reactor Hazards for the	Lanning, W. A. Horning, R.			indicated but no date	experiment plan/design,	photostatic		
168	HW-32791	Prototype Physical Constants Testing Reactor	L. Dickeman	09/16/1954	Hanford	provided	experiment safety analysis	copy, fair	N	
100	1144-27121	i rototype r nysicai constants resting redctor	L. DICKETHAN	03/10/1334	Haillolu	provided	experiment safety analysis	сору, тап	IN	
				1						Pages 2 and 3 are missing from this copy
		Completion of France 11 121 1 111				Connet deal 10 1		L		of the report. Various formulas for
4	25020	Correlation of Exponential Pile Lattice	5 B OL 1 5 5 5 7	02/05/:		Secret, declassified	computational method/data	report, original,		lattice/reactor parameters are presented
169	HW-35038	Measurements with Theory	E. D. Clayton, C. R. Richey	02/08/1955	Hanford	09/29/1955	(1)	good	N	and compared to measured values.
		Exponential Pile Measurements with Hollow				Secret, declassified		report, original,		
170	HW-36174	Slugs in Graphite-Uranium Lattices	E. D. Clayton	04/12/1955	Hanford	06/24/1960	experimental criticality data	good	N	
										For solution process equipment not
										designed to be subcritical under
										personnel reflection, a process for use of
										neutron sources and detectors is
										outlined. The process would allow
		A Proposed Nuclear Safety Indicator for Contact				Secret, declassified		report, original,		verification of safe conditions
171	HW-39433	Maintenance Purposes	G. M. Muller, N. Ketzlach	10/10/1955	Hanford	04/06/1960	experimental criticality data	good	N	(concentration) to allow maintenance.
		<del></del>	·			·	·			

		Exponential Pile Measurements in Water						1		T
							ovnorimental criticality data			
		Moderated Lattice with Enriched Uranium Rods;				Convot declaration	experimental criticality data,	roport original		
470	40000	Buckling Calculations for One Per Cent Enriched	5 5 61	04/45/4055		Secret, declassified	computational method/data	report, original,		
172	HW-40930	Uranium-Water Rod Lattices	E. D. Clayton, H. Neumann	01/16/1956	Hanford	04/12/1957	(1)	good	N	
173	HW-41899	Nuclear Safety of Vessels in Arrays	N. Ketzlach	03/13/1956	Hanford	Secret, declassified 08/29/1960	computational method/data (1)	report, original,	N	
1/3	11W-41033	Nuclear Safety of Vessels III Arrays	IV. REIZIGUI	03/13/1330	Hamord	08/23/1300	(1)	good		A portion of the first page (header
										information) was (deliberately) obscured
										in generation of this hard copy, possibly
		Nuclear Safety of Right Elliptic and Right Annular					computational method/data	report, copy,		for classification reasons. All text of
174	HW-43463	Cylinders	N. Ketzlach	06/01/1956	Hanford	Unclassified	(1)	poor	N	report body is present.
1/4	11W-43403	Nuclear Safety Considerations in Reactor Fuels	IV. REIZIGEII	00/01/1330		Officiassified	(±)	report, copy,		report body is present.
175	HW-43579	Processing Plant Design	N. Ketzlach	06/11/1956	Hanford	Unclassified	equipment/process design	good	Υ	
1/3	11W-43373	Physics Research Quarterly Report April, May,	IV. REIZIBEII	00/11/1330	Hamoru	Officiassified	equipment/process design	report, original,		
176	HW-44525	June 1956	Staff	07/25/1956	Hanford	Unclassified	multiple topics, TOC scanned	good	N	
170	11W-44323	Nuclear Physics Research Quarterly Report	Starr	07/23/1330		Officiassified	multiple topics, roc scanned	report, original,		
177	HW-47012	July, August, September 1956	Staff	11/05/1056	Unnford	Unclassified	multiple topics, TOC scanned	1	N	
1//	NV-47012		Stati	11/05/1956	Hanford	Unclassified	multiple topics, TOC scanned	good report, original,	IN	
178	HW-48893	Nuclear Physics Research Quarterly Report	Staff	03/06/1957	Hanford	Unclassified	multiple tenies TOC seemed	1	N	
1/6	NV-40093	October, November, December 1956	Stati	03/06/1937		Unclassined	multiple topics, TOC scanned	good	IN	
470	104/ 50500	Nuclear Physics Research Quarterly Report	C+-#	05/24/4057	fad	I to all a selfice of		report, original,		
179	HW-50598	January, February, March 1957	Staff	05/31/1957	Hanford	Unclassified	multiple topics, TOC scanned	good	N	
										Preliminary PCTR results and
										interpretations are presented. Callihan's
										folder includes notes re k-infinity
										estimates for uranyl nitrate solution at
										low enrichment, a GE plot of k-infinity
		Barrers Barret or Everying at the Batrersia								for 3% enrichment UO <sub>3</sub> as a function of
		Progress Report on Experiments to Determine				Confidential,				H/U ratio, and an un-numbered (un-
		Infinite Multiplication Factors of Enriched UO <sub>3</sub> -				declassified		report, original,		issued?) copy of the final report by H. E.
180	HW-51168	H <sub>2</sub> O Mixtures	H. E. Handler	07/01/1957	Hanford	01/22/1958	experimental criticality data	good	N	Handler and R. E. Trumble, Jr.
		Nuclear Safety in Processing Uranium Solutions					computational method/data	report, original,		Basic use of a four-factor formula and
181	HW-51364	of All Enrichments	N. Ketzlach	07/15/1957	Hanford	Unclassified	(1)	good	N	age theory to compute criticality data.
		Nuclear Physics Research Quarterly Report						report, original,		
182	HW-51983	April, May, June 1957	Staff	08/14/1957	Hanford	Unclassified	multiple topics, TOC scanned	good	N	
								ř – †		
										Summarizes PCTR measurements to
										determine the minimum critical
		Nuclear Physics Research Quarterly Report July,						report, original,		enrichment of U-235 oxide (~1.03% as
183	HW-53492	August, September 1957	Staff	11/06/1957	Hanford	Unclassified	multiple tenics TOC scanned	good	N	
103	HW-33492	Nuclear Physics Research Quarterly Report	Stail	11/00/1937		Uliciassilleu	multiple topics, TOC scanned	report, original,	IN	UO <sub>3</sub> ) homogeneously mixed with water.
104	LIVAY E 4 E 0.1		Choff	02/05/1050	Unnford	Unalessified	multiple tenies TOC seemed	1	NI.	
184	HW-54591	October, November, December 1957	Staff	03/05/1958	Hanford	Unclassified	multiple topics, TOC scanned	good	N	
405	104/ 55070	Nuclear Physics Research Quarterly Report	C+-ff	04/20/4050	fad	I to all a selfice of		report, original,		
185	HW-55879	January, February, March 1958	Staff	04/29/1958	Hanford	Unclassified	multiple topics, TOC scanned	good	N	
							experimental criticality data,	ropost salainal		
100	LIM 56422	Nuclear Safety of Iron Francis State State	N. Katalash	06/17/1050	Honf	Lindossif:	computational methods/data	report, original,		
186	HW-56423	Nuclear Safety of Iron-Encased Fuel Elements	N. Ketzlach	06/17/1958	Hanford	Unclassified	(1)	good	N	
407	INA 50040	Nuclear Physics Research Quarterly Report	Ct-ff	07/24/4050	Hanfand		TOC	report, original,		
187	HW-56919	April, May, June 1958	Staff	07/21/1958	Hanford	Unclassified	multiple topics, TOC scanned	good	N	Constitution and the second se
4	57064	Nuclear Physics Research Quarterly Report July,	c. "	40/20/5555				report, copy,		Copy includes only section "Critical Mass
188	HW-57861	August, September 1958	Staff	10/20/1958	Hanford	Unclassified	multiple topics	Tair .	N	Physics".
4	50040	Nuclear Safety in Processing Less Than 5.0% U-						report, copy,		
189	HW-58049	235 Enriched Reactor Fuels	N. Ketzlach	11/11/1958	Hanford	Unclassified	equipment/process design	fair	N	
								report, copy,		
190	HW-59066	The Nuclear Safety of Fissile Materials	E. D. Clayton	02/11/1959	Hanford	Unclassified	equipment/process design	fair	Υ	
		Physics Research Quarterly Report October,	-					report, copy,		Copy includes only section "Critical Mass
191	HW-59126	November, December 1958	Staff	01/20/1959	Hanford	Unclassified	multiple topics	fair	N	Physics" and the Reference list.
							1			Considers possible reduction in critical
							facility/process/storage	report, original,		mass requirement due to random
192	HW-59301	Random Loading of E-Metal Dissolver	N. Ketzlach	02/25/1959	Hanford	Unclassified	analysis	good	N	packing of LEU metal slugs in water.
		Nuclear Physics Research Quarterly Report						report, copy,		Copy includes only section "Critical Mass
193	HW-60220	January , February, March 1959	Staff	04/20/1959	Hanford	Unclassified	multiple topics	fair	N	Physics" .
		Nuclear Physics Research Quarterly Report						report, copy,		Copy includes only section "Critical Mass
194	HW-61181	April, May, June 1959	Staff	07/29/1959	Hanford	Unclassified	multiple topics	fair	N	Physics" .

		Nuclear Physics Research Quarterly Report						report, copy,		Copy includes only section "Physics of
195	HW-62727	July, August, September 1959	Staff	10/20/1959	Hanford	Unclassified	multiple topics	fair	N	Nuclear Safety".
		Nuclear Physics Research Quarterly Report						report, original,		·
196	HW-64866	January , February, March 1960	Staff	04/20/1960	Hanford	Unclassified	multiple topics, TOC scanned	good	N	
										Comments on a report by W. Schüller
								report, copy,		titled "The Reactivity of Hetergeneous-
197	HW-65327	Comments on Eurochemic Technical Report 36	E. D. Clayton	05/24/1960	Hanford	Unclassified		fair	N	Homogeneous Systems".
		Nuclearly Safe Mass Limits, Volume Limits,								
		Infinite Cylinder Diameters and Slab Thicknesses								
		for Slightly Enriched Uranium Rods in Light					computational methods/data	report, copy,		
198	HW-65328	Water	E. D. Clayton	05/24/1960	Hanford	Unclassified	(1), equipment/process design	1	N	
			W. A. Reardon, E. D.							
			Clayton, C. L. Brown, R. H.							
			Masterson, T. J. Powell, C.							
		Hazards Summary Report for the Hanford	R. Richey, R. B. Smith, J.					report, original,		
199	HW-66266	Plutonium Critical Mass Laboratory	W. Healy	08/01/1960	Hanford	[None]	experiment safety analysis	good	Υ	
		Hazards Summary Report for the Hanford	C. R. Richey, E. D. Clayton,							
		1	R. H. Odegaarden, J. D.			Unrestricted		report, original,		
200	HW-66266 SUP1 REV	No. 1 The Remote Split-Table Machine	White, W. A. Reardon	10/1963	Hanford	distribution	experiment safety analysis	good	Υ	
		Nuclear Physics Research Quarterly Report	,					report, copy,		Copy includes only section "Critical Mass
201	HW-66215	April, May, June 1960	Staff	07/20/1960	Hanford	[None]	multiple topics	fair	N	Physics".
				1 . , , ,		1		† · · · ·		Results based on PCTR measurements.
		k <sub>∞</sub> of Three Weight Per Cent U <sup>235</sup> Enriched UO <sub>3</sub>	V. I. Neeley, J. A. Berberet,				computational method/data	report, original,		Referenced in footnote of ANSI/ANS-8.1-
202	HW-66882	and UO <sub>2</sub> (NO <sub>3</sub> ) <sub>2</sub> Hydrogenous Systems	R. H. Masterson	09/1961	Hanford	[None]	(1), process/equipment design	good	Υ	1998.
		Measurement of the Negative $k_{ex}$ for a Graphite				[	(2), p. occos, equipment design	F		
								report, original,		Technique for measuring k <sub>∞</sub> in the PCTR
203	HW-67179	Uranium Lattice in the PCTR	E. Z. Block, D. E. Wood	12/01/1960	Hanford	[None]	experimental criticality data	good	N	when the value is less than unity.
										Interest is in the possibility for detecting
		Leakage of Neutrons from Bare Subcritical						report, original,		hazardous (i.e., slightly subcritical)
204	HW-67691	Plutonium Systems	B. C. Clark, J. L. Carter	12/05/1960	Hanford	[None]	computational method (1)	good	N	systems in process environments.
		Physics Research Quarterly Report October,						report, copy,		The copy consists of only the title page
205	HW-68389	November, December 1960	Staff	01/20/1961	Hanford	[None]	multiple topics	fair	N	and table of contents.
		A Semi-Empirical Method of Estimating Material								
		Bucklings for Slightly Enriched Uranium-Water					experimental criticality data,	report, original,		
206	HW-68405	Lattices	C. L .Brown	02/1961	Hanford	[None]	computational method (1)	good	N	
										Good outline of a series of processing
										steps for plutonium fuel element
										manufacture, criticality data, and
										application of criticality controls. Also
										outlines needed data that form the
		Nuclear Safety in Chemical and Metallurgical						report, original,		objectives of the to-be-started Hanford
207	HW-68929	Processing of Plutonium	E. D. Clayton	04/1961	Hanford	[None]	equipment/process design	good	Υ	Pu Critical Mass Laboratory.
		Calculated Critical Parameters for Slightly					experimental criticality data,	report, original,		
208	HW-69273	Enriched Uranium Rods in Light Water	C. L. Brown	04/1961	Hanford	[None]	computational method (1)	good	N	
										The copy consists of only the title page,
										table of contents, and one article
										regarding PCTR k <sub>∞</sub> measurements for low
		Physics Research Quarterly Report January,						report, copy,		enriched (3%) uranium oxide with
209	HW-69475	February, March 1961	Staff	04/20/1961	Hanford	[None]	multiple topics	fair	N	hydrogen moderation.
		Material Buckling Measurements on Graphite-								Over 300 subcritical measurement
		Uranium Systems at Hanford: A Summary						report, original,		configurations are described in detail,
210	HW-69525	Tabulation	D. E. Wood	05/1961	Hanford	[None]	experimental criticality data	good	Υ	several good-quality photos included.
		Measurement of Multiplication Constant for						[		
		Slightly Enriched Homogeneous UO <sub>3</sub> -Water								Analysis of PCTR measurements.
		Mixtures and Minimum Enrichment for	V. I. Neeley and H. E.					report, original,		Referenced in footnote of ANSI/ANS-8.1-
211	HW-70310	Criticality	Handler	08/21/1961	Hanford	[None]	experimental criticality data	good	Υ	1998.
	1144-10310	Circulaty	randici	00/21/1701	i iai il Ulu	[INOTIC]	experimental criticality udid	БООО	'	The copy consists only of the title page
		Physics Research Quarterly Penert April Many						report conv		and one article regarding use of pulsed
242	LIM 70716	Physics Research Quarterly Report April, May,	Chaff	07/20/4064	Honf	[None]	avagrim antal:tilit d	report, copy,	,	
212	HW-70716	June 1961	Staff	07/20/1961	Hanford	[None]	experimental criticality data	fair	N	neutron sources to infer k <sub>eff</sub> .
										The copy consists only of the title page
										and articles from the section titled
										"Critical Mass Physics." Summarizes the
		Physics Research Quarterly Report July,						report, copy,		first experiments performed at the
213	HW-71747	August, September 1961	Staff	10/13/1961	Hanford	[None]	multiple topics	fair	N	Hanford Pu Critical Mass Lab.

										The copy consists only of the title page
		Physics Research Quarterly Report October,						report, copy,		and articles from the section titled
214	HW-72586	November, December 1961	Staff	01/31/1962	Hanford	[None]	multiple topics	fair	N	"Critical Mass Physics."
										These copies appear to be pre-prints of
										individual articles to be included in HW-
										75228. May have been advance copies
		Physics Research Quarterly Report July,						preprint, copy,		provided directly to Callihan by Hanford
215	HW-75228	August, September 1962	Staff	10/15/1962	Hanford	[None]	multiple topics	fair	N	staff.
										Preprint of an article to be included in
										HW-76128, involving Pu solution
										experiments in 14-inch diameter
										spheres. Includes cover letter from E. D.
		Physics Research Quarterly Report January,						preprint, copy,		Clayton to A. D. Callihan dated April 4,
216	HW-76128	February, March 1963	Staff	04/1963	Hanford	[None]	experimental criticality data	good	N	1963.
		The Limiting Critical Concentrations for Pu <sup>239</sup> and	R. H. Masterson, J. D.			Unrestricted		report, original,		
217	HW-77089	U <sup>235</sup> in Aqueous Solutions	White, T. J. Powell	03/27/1963	Hanford	distribution	experimental criticality data	good	Υ	
		Physics Research Quarterly Report April, May,						preprint, copy,		Preprints of 3 articles to be included in
218	HW-77871	June 1963	Staff	07/1963	Hanford	[None]	multiple topics	fair	N	HW-77871.
		Physics Research Quarterly Report July, August,				·	ii	preprint, copy,		Preprint of 1 article to be included in HW
219	HW-79054	September 1963	Staff	10/1963	Hanford	[None]	multiple topics	good	N	79054.
		Physics Research Quarterly Report October,						report, copy,		
220	HW-80020	November, December 1963	Staff	01/15/1964	Hanford	[None]	multiple topics, TOC scanned	good	N	
		Physics Research Quarterly Report April, May,		, , , ,				preprint, copy,		Preprints of 2 articles to be included in
221	HW-83187	June 1964	Staff	07/1964	Hanford	Unclassified	multiple topics	good	N	HW-83187.
				,				report, copy		
		Criticality and Nuclear Safety of Slightly Enriched						from microcard,		
222	HW-SA-2750	Uranium	E. D. Clayton, C. L. Brown	09/11/1962	Hanford	[None]	handbook	fair	N	
			,	, , ,				conference		
								paper, copy		Presented at the 14th National
		Nuclear Safety Standardization at the						from microcard,		Conference on Standards, Washington
223	HW-SA-3310	International Level	M. C. Leverett	01/02/1964	Hanford	Unclassified	standards	fair	N	DC, Feb. 17-19, 1964.
				,,						See Notes on NEPA-1769 re: availability,
224	IC-51-2-7	Calculation, ANP Critical Assembly	AO Mooneyham	2/6/51	AEC/ANP	Unknown				also in T-CSIRC,Vol-1A
										Cover memo w English translation of a
										Mexican report. Multiple fatalities in a
										family resulted from inadvertent
										presence of a 5 curie Co <sup>60</sup> source which
		Report on Radiation Accident, Mexico,	International Commission		Mexican National			memo, copy,		remained in the residence for almost
225	ICRP/63/MC-27	November 1962	on Radiological Protection	06/21/1963	Nuclear Commission	[None]		fair	N	four months.
223	ICIT / 05/IVIC 27	November 1902	on nadiological riotection	00/21/1505	Phillips Petroleum	[reone]		iuii	.,	Use of a reactor-generated neutron
					Company, Idaho Falls,			report, original,		beam to induce fissions in MTR fuel
226	IDO-16114	A Non-Destructive Method for Fuel Assaying	S. G. Forbes	09/22/1953	ID	Unclassified		good	N	elements and infer the fissile content.
	100 10114	A Non Destructive Metrod for Fact Assaying	5. 0. 101003	03/22/1333	10	Officiassifica		good		cientenes and inter the rissue content.
					Phillips Petroleum	Confidential,		report, negative		
		Criticality Calculation with Regard to the			Company, Idaho Falls,		computational method/data	photostatic		
227	IDO-16280	Dissolution of 2% Enriched Slugs	J. W. Webster	06/22/1955	ID	07/29/1960	(1), equipment/process design	copy, fair	N	
221	100-10280	Dissolution of 2/6 Enficined Stags	J. VV. VVEDSLEI	00/22/1333	Phillips Petroleum	07/23/1300	(1), equipment, process design	сору, тап	14	
		A Correlation of Interaction Data with			Company, Idaho Falls,		computational method/data	report, original,		
228	IDO-17095	Calculations Using the Geller Model	J. K. Fox	08/1965	ID			1 1	N	Method is similar to solid-angle method.
440	150-17033	Calculations osing the deliet Model	J. N. 1 UA	00/1503	- JU	[None]	(1)	good	i N	wicehou is similar to soliu-aligie methou.
		A Comparison of Simple Interaction Theory with	R. T. Ackroyd, E. J. Burton,				computational method/data	report, copy,		
229	I G Report 142 (RD/R)	Experiment	M. A. Perks	07/29/1959	U.K.A.E.A.	[None]	(1), equipment/process design	1	N	
	I G NEPOIL 142 (ND/N)	LAPETITIETT	IVI. A. FEIKS	37/23/1339	U.N.A.E.A.	Not to be	(±), equipment/process design	5000		-
						communicated to				
			M. A. Perks, F. R.			any person not				
			Charlesworth, D. E. J.			authorized to receive	computational method/data	report, original,		
220	I G Mama 244 (BD/D)	Criticality of Blutanium Matal Matas Cust		no data	II V A F A	i+		1	N.I	
230	I. G. Memo 244 (RD/R)	Criticality of Plutonium Metal-Water Systems	Thornton	no date	U.K.A.E.A.	Not to be	(1), equipment/process design	goou	N	
						communicated to				
								roport pogative		
		Criticality Data for Homogeneous Uranium-				any person not authorized to receive	computational method/data	report, negative photostatic		
224	IG Mamorandum 463 (BD /B	,	M A Borks	06/10/1959	II V A F A	i+	1	1 1	N.I	
231	io iviemorandum 462 (RD/R	) Water Solutions and Mixtures	M. A. Perks	00/10/1959	U.K.A.E.A.	IL.	(1), equipment/process design	сору, тап	N	

						T		T		
						Not to be				
						communicated to		1		
						any person not				
		Proposed Safety Factor to Allow for				authorized to receive	computational method/data	report, original,		
232	IGR-TN/R.421	Heterogeneity of Fissile Solutions	M. A. Perks	11/1956	U.K.A.E.A.	it.	(1), equipment/process design	good	N	
					Idaho Nuclear					
		Comparison of ORNL Clean Critical Experiments			Corporation, Idaho		computational method/data	report, original,		
233	IN-1120	with Calculations	J. K. Fox	09/1967	Falls, ID	[None]	(2)	good	N	
		Criticality Aspects of the Revised Zirconium			Idaho Nuclear					
		Dissolution System at the Idaho Chemical			Corporation, Idaho		computational method/data	report, original,		
234	IN-1173	Processing Plant	W. G. Morrison	02/1968	Falls, ID	[None]	(2), equipment/process design	good	Υ	
								report, copy		
		Calculated Critical Parameters of Low			Isochem, Inc.,		computational method/data	from micocard,		
235	ISO-174	Enrichment UNH and UO <sub>3</sub> -H <sub>2</sub> O Mixtures	K. R. Ridgway	02/01/1966	Richland WA	Official Use Only	(2)	good	N	
		Calculated Nuclear Safety Parameters of Low			Isochem, Inc.,	+	computational method/data	report, original,		
236	ISO-SA-4	Enriched Uranium	K. R. Ridgway	05/17/1966	Richland WA	Unclassified	(2)	good	N	
230	130-3A-4	Ellicited Oralium	K. N. Mugway	03/17/1300	Michiana WA	Officiassified	(2)	good		Presentation of alpha, beta and gamma
		Radioactivity Hazards Precautions and						report, original,		hazards of uranium, safety guidance and
227	V 27	1	C Viener C Book	07/22/1047	у эг	Destricted		1	N	
237	K-27	Instructions	S. Visner, C. Beck	07/23/1947	K-25	Restricted		fair	N	plant policy
			C. K. Beck, A. D. Callihan,	/ /		Secret, declassified				
238	K-126	Critical Mass Studies, Part II	R. L. Murray	01/23/1948	K-25	08/1957	experimental criticality data		Y	FOLDER IS EMPTY
										Describes manufacture of fissile material
										for the experiments described in K-126,
										plus tolerance measurements. Includes
			W. L. Maroney, B. J.							original (hand-written) 9-page
		Methods for the Preparation of Cubes	Massey, J. G. Schaffner, E.							fabrication procedure, 1-page (hand-
		Containing Uranium Compounds and the	Staple, E. A. Wiggin, A. D.			Secret, declassified		report, original,		written) cost estimate for fabrication
239	K-240	Recovery of the Uranium Salts	Callihan, S. Visner	07/07/1948	K-25	06/25/1958	experimental criticality data	fair	Υ	and material recovery.
			G. B. Knight, A. S. Golin, P.					report, original,		
240	K-317	U-234 Specific Alpha Activity	A. Macklin, R. L. Macklin	12/08/1948	K-25	[None]		fair	N	
										Much of the data requested by Hanford
		Feasibility Study of Uranium Experiments for	C. K. Beck, A. D. Callihan,			Secret, declassified		report, original,		was determined in the K-643 experiment
241	K-320	Obtaining Data Needed in Hanford Operations	E. Grueling, J. W. Morfitt	04/19/1949	K-25	06/25/1958	experiment plan/design	good	N	effort.
			C. K. Beck, A. D. Callihan, J.	<del></del>		Secret, declassified		report, original,		File in includes A. D. Callihan's personal
242	K-343	Critical Mass Studies, Part III	W. Morfitt, R. L. Murray	04/19/1949	K-25	05/08/1958	experimental criticality data	good	Υ	notes regarding the experiments.
	111111111111111111111111111111111111111			- , - , - , - , - , - , - , - , - , - ,				8		Assumes a nearly fully 55 gallon drum of
										solution, most of water in drum must
1						Secret, declassified		report, original,		evaporate to terminate reaction.
243	K-375	Radiation Dosage from Chain Reactions	M. C. Edlund, S. Visner	03/23/1949	K-25	01/22/1957	criticality accident	good	N	Predicts a 500 rad dose radius at 215 ft.
243	IK-373	Radiation bosage from Chain Reactions	D. Callihan, D. F. Cronin, J.	03/23/1343	K-25	01/22/1337	criticality accident	good		rredicts a 300 rad dose radius at 213 rt.
			K. Fox, R. L. Macklin, J. W.			Secret, declassified		report, original,		
244	K-406	Critical Mass Studies, Part IV	Morfitt	11/28/1949	K-25	01/13/1958	experimental criticality data	good	Υ	
	K-400	Citical Mass Studies, Fait IV	WOTTE	11/20/1949	N=23	01/13/1336	experimental criticality data	good		Immediately following this hanging file
										folder is an unlabelled folder, contain
								1		primarily notes and calculations by
ŀ			0.0111 0.50			6		1		Callihan of the reported H/U-235 atom
			D. Callihan, D. F. Cronin, J.			Secret, declassified		report, original,		ratios for the K-643 and other
245	K-643	Critical Mass Studies, Part V	K. Fox, J. W. Morfitt	06/30/1950	K-25	01/13/1958	experimental criticality data	good	Υ	experiments.
			D. Callihan, J. D.			Secret, declassified		report, original,		
246	K-644	Criticality Test on P-10 Alloy Slugs	McClendon, J. W. Morfitt	07/11/1950	K-25	08/04/1960	experimental criticality data	good	Y	
1						Issued as Official Use		report, original,		
						Only, OUO markings		good but		
						stamped over and		several sections		
1						"declassified" stamps	computational method/data	of text have		
247	K-666	Effect of Interaction on Critical Mass	R. L. Macklin	08/31/1950	K-25	added.	(1)	been cut out.	N	
l		A Proposed Method of Evaluating Critical Mass		<b>†</b>		Secret, declassified	computational method/data	report, original,		
248	K-709	Hazards at Low Assays	R. L. Macklin	12/27/1950	K-25	06/24/1960	(1)	good	N	
		,-		1	-	1		<del> </del>		Early development work for criticality
										accident dosimeters using indium foil
		The Energy Spectrum of the Leakage Neutrons				Secret, declassified		report, original,		irradiations performed as part of K-343
249	K-736	from a Homogeneous Reactor	R. C. Rohr, H. F. Henry	07/05/1951	K-25	03/12/1956	dosimetry	good	N	experiments.
243	1. 730	in oni a riomogeneous neactor	n. c. nom, n. r. nemy	01/03/1331	N=23	03/12/1330	иозинен у	boou	IN	елрениена.

	T							1		INTERSECTION OF THE INTERS
										cylinders of unmoderated UF <sub>6</sub> at 2%
										enrichment. Folder includes drawings of
			A. D. Callihan, D. F. Cronin,							
		A Test of Neutron Multiplication by Slightly	R. L. Macklin, J. W.			Secret, declassified		report, original,		tank, cylinders for UF <sub>6</sub> , Cd control blade,
250	K-740	Enriched Uranium	Morfitt, D. V. P. Williams	03/28/1951	K-25	01/21/1959	experimental criticality data	good	N	material analyses.
		Cylindrical Reactor Dimensions of the Water								Hyperbolic curve fit relationships for
		Tamped Enriched Uranyl Fluoride-Water System				Secret, declassified	computational method/data	report, original,		height and diameter of critical solution
251	K-905	as a Function of Concentration	R. L. Macklin	05/09/1952	K-25	01/22/1958	(1)	good	N	cylinders.
		Criticality Data and Nuclear Safety Guide						ľ		· · · · · · · · · · · · · · · · · · ·
		Applicable to the Oak Ridge Gaseous Diffusion	H. F. Henry, A. J. Mallet, C.					report, original,		
252	K-1019 5th Revision	Plant	E. Newlon, W. A. Pryor	05/22/1959	K-25	[None]	equipment/process design	good	Υ	
232	K-1013 Stil Kevision	riant	L. NEWION, W. A. FTYO	03/22/1333	K-25	Secret, declassified	computational method/data	report, original,		
252				00/40/4054		· ·		1 1		
253	K-1141	Water Boiler Calculations of Critical Parameters	H. F. Henry, C. E. Newlon	08/13/1954	K-25	01/22/1958	(1)	good	N	
						Confidential, declassified	computational method/data	report, original,		
254	K-1260	Criticality Calculations for Hydrogenous Systems	J. R. Knight	11/25/1955	K-25	01/22/1958	(1)	good	N	
						Confidential,				
		General Application of a Theory of Neutron	H. F. Henry, J. R. Knight, C.			declassified	computational method/data	report, original,		
255	K-1309	Interaction	W. Newlon	12/15/1956	K-25	12/16/1957	(1)	good	N	
										Folder includes notes and graphs
		A Special Purpose Slide Rule for Computing Solid					computational method/data	report, original,		regarding application of solid angle
256	K-1345	Angles	J. R. Knight	12/19/1957	K-25	Unclassified	(1)	good	N	methodology.
230	IX 1343	Extension of the Safe Geometric Parameters to	J. IV. IVIIBIIL	12/13/1337	K-23	Officiassificu	computational method/data	report, original,		methodology.
257	K 1270	1	C E Namian	11/22/1050	к эг	Unclassified		1		
257	K-1370	Slightly Enriched Uranium	C. E. Newlon	11/23/1958	K-25	Unclassified	(1)	good	N	
350	K 1300			00/44/4050	v 25	Undersitänd	handhad.	report, original,	v	Lectures presented at the Nuclear Safety
258	K-1380	Studies in Nuclear Safety	H. F. Henry (Compiler) K. W. Bahler, A. F. Rupp, R.	08/14/1958	K-25	Unclassified	handbook	good	Υ	Training School, June 3 - 14, 1957.
			H. Lafferty Jr., J. W.							May be viewed as a comprehensive
i		Prevention and Handling of Radiation	Wachter, L. S. O'Rourke, C.					report, original,		lessons-learned from the 1958 Y-12
259	K-1436	Emergencies	R. Milone	12/22/1959	K-25	Unclassified	criticality accident	good	Υ	accident.
					-			report, original,		
260	K-1470	Is All Radiation Harmful?	H. F. Henry	05/02/1961	K-25	Unclassified		good	N	
			H. F. Henry, C. E. Newlon,				computational method/data	report, original,		
261	K-1478	Extensions of Neutron Interaction Data	J. R. Knight	07/11/1961	K-25	Unclassified	(1)	good	N	Solid angle methodology
										Applies various nuclear analysis methods
								1		to the PCTR measurements; established
										a subcritical enrichment limit of 0.95
		The Effect of Uranium Density on the Cof- 11 225					computational mathad/d-t-	roport crisinal		
202	K 4550	The Effect of Uranium Density on the Safe U-235	C E Name	40/44/4052	W 25	Line alone (C)	computational method/data	report, original,		weight % U-235 for the Oak Ridge
262	K-1550	Enrichment Criterion	C. E. Newlon	10/11/1962	K-25	Unclassified	(1), equipment/process design	-	N	Diffusion Plant.
		A Nuclearly Safe 12-1/14-In. I.D. Packed Liquid		[ [				report, original,		
263	K-1582	Entrainment Separator	H. F. Henry, C. E. Newlon	10/17/1963	K-25	[None]	equipment/process design	good	N	
										Notes that for equal-volume arrays of
										individually subcritical air-spaced units,
1								1		minimum critical array U-235 mass
		The Elements of Neutron Interacting Arrays -					computational method/data	report, original,		occurs at enrichments substantially less
264	K-1619	Part 1	C. E. Newlon	10/13/1964	K-25	[None]	(1), equipment/process design	1	N	than 93% enrichment.
<del></del>				,,,	25		, ,, ==================================	†*****************************		Uses experiment results and 16-group
1								1		transport calculations. Concludes that
1		Asia in the Control Collins I Collin						L		-
	W 4520	Minimum Critical Cylinder Diameters of	0.5.11	00/45/4555		fa. 1	computational method/data	report, original,		the minimum critical diameter for ~ 5%
265	K-1629	Hydrogen Moderated U(4.9) Systems	C. E. Newlon	03/15/1965	K-25	[None]	(1), equipment/process design	good	N	enrichment is ~ 12 inches.

266   Sc. 1490   Sec.   Sc. 1490   Sec.	268 269 270 271 272	K-1661 K-1663 K-1669 K-1714 K-1716	Report Fiscal Year 1964 Through Fiscal Year 1965 Fissile-Material Container and Packaging Development and Testing Program Hydrogen Moderation - A Primary Nuclear Safety Control for Handling and Transporting Low- Enrichment UF <sub>6</sub> ORGDP Reprocessing Studies Summary Progress Report July Through December, 1965 Protective Shipping Packages for 8- and 12-inch Diameter UF <sub>6</sub> Cylinders Protective Shipping Package for 5-inch Diameter UF <sub>6</sub> Cylinder Nuclear Safety Studies for Low Enrichment Fluoride Volatility Fuel Reprocessing Plants	H. Pashley A. J. Mallett, S. J. Wheatley C. E. Newlon, A. J. Mallett S. H. Smiley, D. C. Brater, J. H. Pashley A. J. Mallett, C. E. Newlon	04/01/1966 05/31/1966 06/28/1966	K-25 K-25 K-25	[None]	transport safety analysis	good report, original, good report, original, good	N	Reprocessing via fluoride volatility methods are assessed from a variety of engineering standpoints, including criticality safety. Flow diagrams, equipment designs, etc. addressed.  Design efforts for UF <sub>6</sub> transport containers.
March   Marc	268 269 270 271 272	K-1661 K-1663 K-1669 K-1714 K-1716	Report Fiscal Year 1964 Through Fiscal Year 1965 Fissile-Material Container and Packaging Development and Testing Program Hydrogen Moderation - A Primary Nuclear Safety Control for Handling and Transporting Low- Enrichment UF <sub>6</sub> ORGDP Reprocessing Studies Summary Progress Report July Through December, 1965 Protective Shipping Packages for 8- and 12-inch Diameter UF <sub>6</sub> Cylinders Protective Shipping Package for 5-inch Diameter UF <sub>6</sub> Cylinder Nuclear Safety Studies for Low Enrichment Fluoride Volatility Fuel Reprocessing Plants	H. Pashley A. J. Mallett, S. J. Wheatley C. E. Newlon, A. J. Mallett S. H. Smiley, D. C. Brater, J. H. Pashley A. J. Mallett, C. E. Newlon	04/01/1966 05/31/1966 06/28/1966	K-25 K-25 K-25	[None]	transport safety analysis	good report, original, good report, original, good	N	engineering standpoints, including criticality safety. Flow diagrams, equipment designs, etc. addressed.  Design efforts for UF <sub>6</sub> transport
MRDP Representing Basiles Sammary Frogenses   K. Smilley, D. C. Bardy   Special Control for 1928 of 1929 for	268 269 270 271 272	K-1661 K-1663 K-1669 K-1714 K-1716	Report Fiscal Year 1964 Through Fiscal Year 1965 Fissile-Material Container and Packaging Development and Testing Program Hydrogen Moderation - A Primary Nuclear Safety Control for Handling and Transporting Low- Enrichment UF <sub>6</sub> ORGDP Reprocessing Studies Summary Progress Report July Through December, 1965 Protective Shipping Packages for 8- and 12-inch Diameter UF <sub>6</sub> Cylinders Protective Shipping Package for 5-inch Diameter UF <sub>6</sub> Cylinder Nuclear Safety Studies for Low Enrichment Fluoride Volatility Fuel Reprocessing Plants	H. Pashley A. J. Mallett, S. J. Wheatley C. E. Newlon, A. J. Mallett S. H. Smiley, D. C. Brater, J. H. Pashley A. J. Mallett, C. E. Newlon	04/01/1966 05/31/1966 06/28/1966	K-25 K-25 K-25	[None]	transport safety analysis	good report, original, good report, original, good	N	criticality safety. Flow diagrams, equipment designs, etc. addressed. Design efforts for UF <sub>6</sub> transport
	268 269 270 271 272	K-1661 K-1663 K-1669 K-1714 K-1716	Report Fiscal Year 1964 Through Fiscal Year 1965 Fissile-Material Container and Packaging Development and Testing Program Hydrogen Moderation - A Primary Nuclear Safety Control for Handling and Transporting Low- Enrichment UF <sub>6</sub> ORGDP Reprocessing Studies Summary Progress Report July Through December, 1965 Protective Shipping Packages for 8- and 12-inch Diameter UF <sub>6</sub> Cylinders Protective Shipping Package for 5-inch Diameter UF <sub>6</sub> Cylinder Nuclear Safety Studies for Low Enrichment Fluoride Volatility Fuel Reprocessing Plants	H. Pashley A. J. Mallett, S. J. Wheatley C. E. Newlon, A. J. Mallett S. H. Smiley, D. C. Brater, J. H. Pashley A. J. Mallett, C. E. Newlon	04/01/1966 05/31/1966 06/28/1966	K-25 K-25 K-25	[None]	transport safety analysis	good report, original, good report, original, good	N	equipment designs, etc. addressed.  Design efforts for UF <sub>6</sub> transport
Paulie National Continues and Packaging (Mark National	268 269 270 271 272	K-1661 K-1663 K-1669 K-1714 K-1716	Fissile-Material Container and Packaging Development and Testing Program Hydrogen Moderation - A Primary Nuclear Safety Control for Handling and Transporting Low- Enrichment UF <sub>6</sub> ORGDP Reprocessing Studies Summary Progress Report July Through December, 1965 Protective Shipping Packages for 8- and 12-Inch Diameter UF <sub>6</sub> Cylinders Protective Shipping Package for 5-Inch Diameter UF <sub>6</sub> Cylinder Nuclear Safety Studies for Low Enrichment Fluoride Volatility Fuel Reprocessing Plants	A. J. Mallett, S. J. Wheatley C. E. Newlon, A. J. Mallett S. H. Smiley, D. C. Brater, J. H. Pashley A. J. Mallett, C. E. Newlon	04/01/1966 05/31/1966 06/28/1966	K-25 K-25 K-25	[None]	transport safety analysis	report, original, good report, original, good	N	Design efforts for UF <sub>6</sub> transport
1.561   Development and Testage Programs   Windstage   Oxfort (1975)   Oxfor	268 269 270 271 272	K-1663 K-1669 K-1714 K-1716	Development and Testing Program  Hydrogen Moderation - A Primary Nuclear Safety Control for Handling and Transporting Low- Enrichment UF <sub>6</sub> ORGDP Reprocessing Studies Summary Progress Report July Through December, 1965 Protective Shipping Packages for 8- and 12-Inch Diameter UF <sub>6</sub> Cylinders Protective Shipping Package for 5-Inch Diameter UF <sub>6</sub> Cylinder  Nuclear Safety Studies for Low Enrichment Fluoride Volatility Fuel Reprocessing Plants	Wheatley C. E. Newlon, A. J. Mallett S. H. Smiley, D. C. Brater, J. H. Pashley A. J. Mallett, C. E. Newlon	05/31/1966 06/28/1966	K-25 K-25	[None]		good report, original,		
\$2.19.2   Development and Festation Programs   Ministry   Development and Festation Programs   Notice Medication - American	268 269 270 271 272	K-1663 K-1669 K-1714 K-1716	Development and Testing Program  Hydrogen Moderation - A Primary Nuclear Safety Control for Handling and Transporting Low- Enrichment UF <sub>6</sub> ORGDP Reprocessing Studies Summary Progress Report July Through December, 1965 Protective Shipping Packages for 8- and 12-Inch Diameter UF <sub>6</sub> Cylinders Protective Shipping Package for 5-Inch Diameter UF <sub>6</sub> Cylinder  Nuclear Safety Studies for Low Enrichment Fluoride Volatility Fuel Reprocessing Plants	Wheatley C. E. Newlon, A. J. Mallett S. H. Smiley, D. C. Brater, J. H. Pashley A. J. Mallett, C. E. Newlon	05/31/1966 06/28/1966	K-25 K-25	[None]		good report, original, good		containers.
Psychogen Noteration - A Primary Noteration State   Control Controlling and Transport Informating and Transport Information   Psychologist   Control Controlling and Transport Information   Psychologist   Control Controlling and Psychologist   Control Contr	268 269 270 271 272	K-1663 K-1669 K-1714 K-1716	Hydrogen Moderation - A Primary Nuclear Safety Control for Handling and Transporting Low- Enrichment UF <sub>6</sub> ORGDP Reprocessing Studies Summary Progress Report July Through December, 1965 Protective Shipping Packages for 8- and 12-Inch Diameter UF <sub>6</sub> Cylinders Protective Shipping Package for 5-Inch Diameter UF <sub>6</sub> Cylinder Nuclear Safety Studies for Low Enrichment Fluoride Volatility Fuel Reprocessing Plants	C. E. Newlon, A. J. Mallett S. H. Smiley, D. C. Brater, J. H. Pashley A. J. Mallett, C. E. Newlon	05/31/1966 06/28/1966	K-25 K-25	[None]		report, original,		containers.
Control for Handling and Transporting Low-   Factorism Hard   Factorism Hard   Control   Contr	269 270 271 272	K-1669 K-1714 K-1716	Control for Handling and Transporting Low- Enrichment UF <sub>6</sub> ORGDP Reprocessing Studies Summary Progress Report July Through December, 1965 Protective Shipping Packages for 8- and 12-Inch Diameter UF <sub>6</sub> Cylinders Protective Shipping Package for 5-Inch Diameter UF <sub>6</sub> Cylinder Nuclear Safety Studies for Low Enrichment Fluoride Volatility Fuel Reprocessing Plants	C. E. Newlon, A. J. Mallett S. H. Smiley, D. C. Brater, J. H. Pashley A. J. Mallett, C. E. Newlon	06/28/1966	K-25		transport safety analysis	good	N	
2-60   Furthermore Use   C. Reedon, A. J. Mallett   C. Sparker, S. Sparker, C. Sparker,	269 270 271 272	K-1669 K-1714 K-1716	Enrichment UF <sub>6</sub> ORGDP Reprocessing Studies Summary Progress Report July Through December, 1965 Protective Shipping Packages for 8- and 12-Inch Diameter UF <sub>6</sub> Cylinders Protective Shipping Package for 5-Inch Diameter UF <sub>6</sub> Cylinder Nuclear Safety Studies for Low Enrichment Fluoride Volatility Fuel Reprocessing Plants	S. H. Smiley, D. C. Brater, J. H. Pashley A. J. Mallett, C. E. Newlon	06/28/1966	K-25		transport safety analysis	good	N	
	269 270 271 272	K-1669 K-1714 K-1716	ORGDP Reprocessing Studies Summary Progress Report July Through December, 1965 Protective Shipping Packages for 8- and 12-Inch Diameter UF <sub>6</sub> Cylinders Protective Shipping Package for 5-Inch Diameter UF <sub>6</sub> Cylinder Nuclear Safety Studies for Low Enrichment Fluoride Volatility Fuel Reprocessing Plants	S. H. Smiley, D. C. Brater, J. H. Pashley A. J. Mallett, C. E. Newlon	06/28/1966	K-25		transport safety analysis	F	N	1
256   Report July Through December, 1985   Pastery   Original Process Supply Records 5 and 12 lated   Report Supply Records 6 and 12 lated   Records 5 and 12 lated   Report Supply Records 6 and 12 lated   Records 6	270 271 272	K-1714 K-1716	Report July Through December, 1965  Protective Shipping Packages for 8- and 12-inch Diameter UF <sub>6</sub> Cylinders  Protective Shipping Package for 5-inch Diameter UF <sub>6</sub> Cylinder  Nuclear Safety Studies for Low Enrichment Fluoride Volatility Fuel Reprocessing Plants	H. Pashley A. J. Mallett, C. E. Newlon			[None]		report, original		
Protective Shipping Packages for 8 and 12 inch   A.J. Mallett, C.E. Revelon   Of/20/1957   K.25   None    equipment/process design, report, original,   Protective Shipping Package for 5-inch Dismoster   A.J. Mallett, C.E. Revelon   Of/20/1957   K.25   None    equipment/process design, report, original,   Protective Shipping Package for 5-inch Dismoster   A.J. Mallett, C.E. Revelon   Of/20/1957   K.25   None    equipment/process design, report, original,   Protective Shipping Package for 5-inch Dismoster   Of/20/1957   K.25   None    equipment/process design, report, original,   Protective Shipping Package for 5-inch Dismoster   Of/20/1957   K.25   None    equipment/process design, report, original,   Protective Shipping Package for 5-inch Dismoster   Of/20/1957   K.25   None    equipment/process design, report, original,   Protective Shipping Package for 5-inch Dismoster   Of/20/1958   K.25   None    equipment/process design, report, original,   Protective Shipping Package for 5-inch Dismoster   Of/20/1958   K.25   None    equipment/process design, report, original,   Protective Shipping Package for 5-inch Dismoster   Of/20/1958   K.25   None    equipment/process design, report, original,   Protective Shipping Package for 5-inch Dismoster   Of/20/1958   K.25   None    equipment/process design, report, original,   Protective Shipping Package for 5-inch Dismoster   Of/20/1958   K.25   None    Of/20/1958   Of/20/	270 271 272	K-1714 K-1716	Protective Shipping Packages for 8- and 12-Inch Diameter UF <sub>6</sub> Cylinders Protective Shipping Package for 5-Inch Diameter UF <sub>6</sub> Cylinder Nuclear Safety Studies for Low Enrichment Fluoride Volatility Fuel Reprocessing Plants	A. J. Mallett, C. E. Newlon			[None]	i .	1 -1,,, 8,		
273   274   Dainete UF, Chimies   A. Mallert, C. E. Newton   Da/20/1997   E. 25   Robe   transport after party analysis   good   N	271 272	K-1716	Diameter UF <sub>6</sub> Cylinders  Protective Shipping Package for 5-Inch Diameter  UF <sub>6</sub> Cylinder  Nuclear Safety Studies for Low Enrichment  Fluoride Volatility Fuel Reprocessing Plants		04/20/1967			equipment/process design	good	Υ	
Potential Symbol   Potential S	271 272	K-1716	Protective Shipping Package for 5-Inch Diameter UF <sub>6</sub> Cylinder Nuclear Safety Studies for Low Enrichment Fluoride Volatility Fuel Reprocessing Plants		04/20/1967			equipment/process design,	report, original,		
K7756	272		UF <sub>6</sub> Cylinder  Nuclear Safety Studies for Low Enrichment Fluoride Volatility Fuel Reprocessing Plants	A. J. Mallett, C. E. Newlon		K-25	[None]	transport safety analysis	good	N	
273   K-738   Ji, Cylinder   A. Mallert, C.E. Newlon   06/43/1967   K-75   None  management part   And   None	272		UF <sub>6</sub> Cylinder  Nuclear Safety Studies for Low Enrichment Fluoride Volatility Fuel Reprocessing Plants	A. J. Mallett, C. E. Newlon					report original		<del></del>
Nuclear Select, Studies for two Interhement	272		Nuclear Safety Studies for Low Enrichment Fluoride Volatility Fuel Reprocessing Plants	A. J. Mailett, C. E. Newion	06/14/1067	V 25	[None]		1	N	
Extraction   Ambient Membrane plant   Extraction   Ambient Membrane		K-1734	Fluoride Volatility Fuel Reprocessing Plants		00/14/1907	K-23	[None]		E	IN	
A Measurement of Iris and Other Fiscolon Parameters for U.23p. 1293, and pp. 421, D. E. McMillian, M. E. Jones, J. B. Sampson, E. R. Jones, J. J. Fitzgerda, R. J. Professor Radiation Energencies Steering Committee Standard Energencies Steering Committee J. W. Baller, J. F. Bapp, R. J. Lafferly J. J. N. J. Laf		K-1734	<del>-</del>						1		
Parameters for U-233, Pu-239, and Pu-241, Malletter N-243 & Louis Confirmant Value for U-234 (Model and New Journal Management of U-234, Pu-239). A Model Manipulator for Use in Controlling A. Millett Model and Controlling A. Model Manipulator for Use in Controlling A. Millett Model Manipulator for Use in Controlling A. Model Manipulator for Use in Controlling Manipulator for Use in Controlling A. Model Manipulator	272		A Measurement of Eta and Other Fission	K. E. Habiger, D. L. Breton	09/04/1968	K-25	[None]	analysis	good	Υ	
Reliable to U.23 at 30-D-Codmunion Neutron   Sones, B. Sampson, E. R.   Sertinger, T. N. Sertinger, T. Sertinger,	272								1 T		
APPL-1846   Energies   Selective, T. M. Snyder   12/15/1935   KAPL   Unclassified   Nuclear measurement/data   good   N	272		Parameters for U-233, Pu-239, and Pu-241,	D. E. McMillan, M. E.							
APL-1464   Sergies   Searther, T. M. Snyder   12/15/1935   KAPL   Unclassified   Nuclear measurement/data   good   N	272		Relative to U-235 at Sub-Cadmium Neutron	Jones, J. B. Sampson, E. R.					report, original,		
Environmental Hazard Evaluations for Critical   1, Fittgereldi, R. J.   12/65/1956   1.   1.   1.   1.   1.   1.   1.   1	2/3	KAPL-1464	j i		12/15/1955	KAPL	Unclassified	nuclear measurement/data	1	N	
APL-1787 Assembles feriberg 12/05/1956 KAPL Unclarible corporation and Marker L. S. O'Rourke, C. Ne, Babler, A. F. Rupp, R. R. Lafferty, F. J. W. Wather, L. S. O'Rourke, C. No/1959 Portromothy of the public orporation and Marker L. S. O'Rourke, C. No/1959 Portromothy without approval. criticality accident report, original, production of the public organization of the pu					, ,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,		+			-	<del> </del>
K. W., Balher, A. F. Rupp, R. H. Lafferty Ir. J. W. Wather, L. S. O'Rourke, C. Wather, L. S. O'Rourke, C. Montes Summary Report  A Mobile Manipulator for Use in Controlling Radiation Emergencies Report organia, Industriation Report organia, Industriati	274	VADI 1707	1	- '	12/05/1056	KADI	Unclassified	ovnoriment cafety analysis	1	N	
Radiation Emergencies Steering Committee  275 R9-751  287 S1 Summary Report  A Mobile Manipulator for Use in Controlling A Mobile Manipulator for Use in Controlling A F. Becher  Oly/05/1965  A F. Bech	2/4	KAFL-1707	Assemblies	remberg	12/03/1930		Uliciassilleu	experiment safety analysis	good		+
Radiation Emergencies Steering Committee  8. H. Lifferty /F. J. W. Worker, L. S. O'Rourke, C. R. Milone  9. Portsmouth)  8. H. J. O'Rourke, C. R. Milone  9. O'A/1959  9. Portsmouth)  Michael Corporation (C5 and external distribution  9. A. F. Becher  9. A. F. Beche									1 1		
Radiation Emergencies Steering Committee  R. Milone  A Mohle Manipulator for Use in Controlling  A Mohle Manipulator for Use in Controlling  Radiation Emergencies  A. F. Becher  A. F.			1						1 1		
Ref.   Summary Report   Summary Report   Ref.   Summary Report   Summary Report   Ref.   Summary Report   Summary Report   Summary Report   Summary Ref.   Summary Report   Summary Rep				H. Lafferty Jr., J. W.		Goodyear Atomic	the public or given		1 1		
A Mobile Manipulator for Use in Controlling Radiation Emergencies			Radiation Emergencies Steering Committee	Wachter, L. S. O'Rourke, C.		Corporation (K-25 and	external distribution		report, original,		
A Mobile Manipulator for Use in Controlling Radiation Emergencies  A F, Becher  A J	275	KB-751	Summary Report	R. Milone	04/1959	Portsmouth)	without approval	criticality accident	good	N	Similar scope/content as K-1436.
A Mobile Manipulator for Use in Controlling Radiation Emergencies A. F. Becher A. J. Mallet A. J											Includes engineering specifications for a
A Mobile Manipulator for Use in Controlling Radiation Foregracies A, F, Becher A, F									1 1		remotely controlled robot, for purpose
Ref.			A Mobile Manipulator for Use in Controlling						report conv		of achieving secure termination of a
Critical Interaction Potentials for Water Moderated UF <sub>6</sub> Arrays  C. E. Newlon  11/30/1962  K. D. 1810  A Triangular Slab Abrasive Slurry Tank for Handling Enriched Uranium  C. E. Newlon  O3/19/1964  K. D. 1810  Approval Letter No. 143 Criteria Change: In- Plant Moderation Controlled UF <sub>6</sub> Cylinders  Nuclear Safety Analysis of Low Enrichment Uranium Oxide Shipment of 30-ran Cylinder Swannah River Plant  R. J. Mallett  O6/33/1965  K. D. 1883  A. J. Mallett  O6/33/1966  K. D. 1883  Conceptual Design Report for Critical Assembly Bay ELA  Conceptual Design Report for Critical Assembly Conceptual Design Report for Critical Assembly Bay ELA  Criteria Change: Protective Structural Package  C. E. Newlon  O3/19/1964  K. 25  [None]  (1)  (1)  (1)  (1)  (1)  (2)  (1)  (1)	276	K-C-768	1	Λ E Becher	04/05/1965	V-25	[None]	criticality accident	1	N	_
277 KD-1766 Moderated UF <sub>ii</sub> Arrays C. E. Newlon 11/30/1962 K25 [None] (1) good N  A Triangular Slab Abrasive Slurry Tank for A Triangular Slab Abrasive Slurry Tank for C. E. Newlon 03/19/1964 K25 [None] (1), equipment/process design good N  Approval Letter No. 143 Criteria Change: In-Plant Moderation Controlled UF <sub>ic</sub> Cylinders  A Decomposite Confidential, Co	270	K-C-700		A. I . Bechel	04/03/1303	K-25	[NOTIC]		L		criticality accident.
A Triangular Slab Abrasive Slurry Tank for Handling Enriched Uranium C. E. Newlon 03/19/1964 K.25 [None] (1), equipment/process design good N cylinder.  Approval Letter No. 143 Criteria Change: In-Plant Moderation Controlled UF <sub>6</sub> (yinders No. Least Safety Analysis Contential, Handling Enriched Uranium Oxide Shipment to Savannah River Plant Uranium Oxide Shipment to Savannah River Plant Uranium Oxide Shipment of 10-Ton Cylinders Containing A. J. Mallett 06/23/1965 K.25 [None] transport safety analysis good N Engineering design for a to be co-constructed with Conceptual Design Report for Critical Assembly Staff (of the Engineering Division) 06/15/1966 K.25 [None] experiment plan/design good N Electron Linear Accelerates Structural Package A. D. Callihan, A. P. Huber, Criteria Change: Protective Structural Package A. D. Callihan, A. P. Huber, Criteria Change: Protective Structural Package A. D. Callihan, A. P. Huber, Criteria Change: Protective Structural Package A. D. Callihan, A. P. Huber, R. L. Clouse, A. J. Mallett C. E. Newlon 03/19/1964 K.25 [None] transport safety analysis good N Engineering good N Electron Linear Accelerates Structural Package A. D. Callihan, A. P. Huber, Criteria Change: Protective Structural Package A. D. Callihan, A. P. Huber, R. L. Clouse, A. J. Mallett C. E. Newlon 03/19/1964 K.25 [None] transport safety analysis good N Electron Linear Accelerates Structural Package A. D. Callihan, A. P. Huber, R. L. Clouse, A. J. Mallett C. C. E. Newlon 03/19/1966 K.25 [None] experiment plan/design good N Electron Linear Accelerates Structural Package A. D. Callihan, A. P. Huber, R. L. Clouse, A. J. Mallett C. Canada Control Contr					/				1		
A Triangular Slab Abrasive Slurry Tank for Handling Enriched Uranium C. E. New, A. J. Mallett, C	2//	KD-1766	Moderated UF <sub>6</sub> Arrays	C. E. Newlon	11/30/1962	K-25	[None]	(1)	good	N	
A Triangular Slab Abrasive Slurry Tank for Handling Enriched Uranium C. E. Newlon O. 3/19/1964 K.25 [None] computational method/data report, original, eport, original, policy for a segment of C. E. Newlon O. 3/19/1964 K.25 [None] confidential, declassified equipment/process design and policy for a segment of C. E. Newlon O. 3/19/1964 K.25 [None] confidential, declassified equipment/process design fair N. Illimits for in-plant UFa, cylinders.  A D. Callihan, A. P. Huber, R. L. Macklin, A. J. Mallet O. 0/(03/1964 K.25 0.02/01/1966 equipment/process design fair N. Illimits for in-plant UFa, cylinders.  A J. Mallett O. 0/(03/1965 K.25 [None] transport safety analysis good N. M. Increased the allowable equipment/process design for a controlled UFa, cylinders.  A J. Mallett O. 0/(03/1965 K.25 [None] transport safety analysis good N. M. Increased the allowable equipment/process design for a controlled UFa, cylinders.  A J. Mallett O. 0/(03/1965 K.25 [None] transport safety analysis good N. M. Increased the allowable equipment/process design for a controlled UFa, cylinders of increased the allowable equipment/process design for a controlled UFa, cylinders of increased the allowable equipment/process design for a controlled UFa, cylinders of increased the allowable equipment/process design for a controlled UFa, cylinders of increased the allowable equipment/process design for a controlled UFa, cylinders of increased the allowable equipment/process design for a controlled UFa, cylinders of increased the allowable equipment/process design for a controlled UFa, cylinders of increased the allowable equipment/process design for a controlled UFa, cylinders of increased the allowable equipment/process design for a controlled UFa, cylinders of increased the allowable equipment/process design for a controlled UFa, cylinders of increased the allowable equipment/process design for a controlled UFa, cylinders of increased the allowable equipment/process design for a controlled UFa, cylinders of increased the allowable equipment/pro									1 1		Has numerous hand-written notes by J.
R-D-1810   Handling Enriched Uranium   C. E. Newlon   O3/19/1964   K-25   None]   (1), equipment/process design   good   N   Oylinders									1 1		T. Thomas. Develops geometric buckling
Approval Letter No. 143 Criteria Change: In- Plant Moderation Controlled UF <sub>6</sub> Cylinders Nuclear Safety Analysis of Low Enrichment Uranium Oxide Shipment to Savannah River Plant Novige Shipment to Savannah River R. L. Macklin, A. J. Mallett O6/23/1965 N. J. Mallett O6/23/1966 N. J. Mallett O6/23			A Triangular Slab Abrasive Slurry Tank for	R. J. Clouse, A. J. Mallett,				computational method/data	report, original,		values for a segment of a right circular
Approval Letter No. 143 Criteria Change: In- Plant Moderation Controlled UF <sub>6</sub> Cylinders Nuclear Safety Analysis of Low Enrichment Uranium Oxide Shipment to Savannah River Plant Rail Shipment of 10-Ton Cylinders Containing 281 K-D-1858 Plant Confidential, declassified Q4/03/1964 R. L. Macklin, A. J. Mallett Q6/23/1965 Q2/01/1966	278	K-D-1810	Handling Enriched Uranium	C. E. Newlon	03/19/1964	K-25	[None]	(1), equipment/process design	good	N	cylinder.
Approval Letter No. 143 Criteria Change: In-Plant Moderation Controlled UF <sub>6</sub> Cylinders  No. 143 Criteria Change: In-Plant Moderation Controlled UF <sub>6</sub> Cylinders  No. 143 Criteria Change: In-Plant Moderation Controlled UF <sub>6</sub> Cylinders  No. 143 Criteria Change: In-Plant Moderation Controlled UF <sub>6</sub> Cylinders  No. 143 Criteria Change: In-Plant Moderation Controlled UF <sub>6</sub> Cylinders  No. 143 Criteria Change: In-Plant Moderation Controlled UF <sub>6</sub> Cylinders  No. 143 Criteria Change: In-Plant Moderation  No. 144 Criteria Change: In-								7, 4, 1, 4, 1, 4, 1, 1, 1, 1, 1, 1, 1, 1, 1, 1, 1, 1, 1,			
Plant Moderation Controlled UF <sub>6</sub> Cylinders Nuclear Safety Analysis of Low Enrichment Uranium Oxide Shipment to Savannah River Plant Nuclear Safety Analysis of Low Enrichment Uranium Oxide Shipment to Savannah River Plant A. J. Mallett O6/23/1965 K-25 [None] Transport safety analysis good N  Engineering Division)  Staff (of the Engineering Division)  Conceptual Design Report for Critical Assembly Bay ELA  Criteria Change: Protective Structural Package  A. D. Callihan, A. P. Huber,  Criteria Change: Protective Structural Package  R. L. Macklin, A. J. Mallett O4/03/1964 K-25 [None]  K-25 [None]  Conceptual Design Report for Critical Assembly Bay ELA  Criteria Change: Protective Structural Package  A. D. Callihan, A. P. Huber,  Criteria Change: Protective Structural Package  Criteria Change: Protective Structural Package  Conceptual Design Report for Critical Assembly Criteria Change: Protective Structural Package  A. D. Callihan, A. P. Huber,  Criteria Change: Protective Structural Package  Criteria C			Approval Letter No. 143 Criteria Change: In-	A D Callihan A P Huber					memo conv		Increased the allowable enrichment
Nuclear Safety Analysis of Low Enrichment Uranium Oxide Shipment to Savannah River Plant Rail Shipment of 10-Ton Cylinders Containing 281 K-D-1893 2.1% Enriched UF <sub>6</sub> A. J. Mallett 01/28/1966 K-25 [None] transport safety analysis good N  Conceptual Design Report for Critical Assembly Bay ELA Division) 06/15/1966 K-25 [None] experiment plan/design good N  Engineering design for a report, original, to be co-constructed with the conceptual Design Report for Critical Assembly Division) 07/29/1966 K-25 [None] experiment plan/design good N  Engineering design for a report, original, good N  Electron Linear Accelerate the conceptual Design Report for Critical Assembly Division) 07/29/1966 K-25 [None] experiment plan/design good N  Engineering design for a report, original, good N  Electron Linear Accelerate the conceptual Design Report for Critical Assembly Division) 07/29/1966 K-25 [None] experiment plan/design good N  Authorizes off-site shipp UF <sub>6</sub> cylinders with up to enrichments. Includes; descriptions of experiment plan/design good N  Criteria Change: Protective Structural Package A. D. Callihan, A. P. Huber, Criteria Change: Protective Structural Package A. D. Callihan, A. P. Huber, Criteria Change: Protective Structural Package A. D. Callihan, A. P. Huber, Criteria Change: Protective Structural Package A. D. Callihan, A. P. Huber, Criteria Change: Protective Structural Package A. D. Callihan, A. P. Huber, Criteria Change: Protective Structural Package A. D. Callihan, A. P. Huber, Criteria Change: Protective Structural Package A. D. Callihan, A. P. Huber, Criteria Change: Protective Structural Package A. D. Callihan, A. P. Huber, Criteria Change: Protective Structural Package A. D. Callihan, A. P. Huber, Criteria Change: Protective Structural Package A. D. Callihan, A. P. Huber, Criteria Change: Protective Structural Package A. D. Callihan, A. P. Huber, Criteria Change: Protective Structural Package A. D. Callihan, A. P. Huber, Criteria Change: Protective Structural Package A. D. Callihan Criteria Change: Prote	270	K-D-1917			04/03/1964	V-25			1	N	
Uranium Oxide Shipment to Savannah River Plant A. J. Mallett O6/23/1965 K-25 [None] Transport safety analysis good N Rail Shipment of 10-Ton Cylinders Containing 2.81 K-D-1893 2.1% Enriched UF <sub>6</sub> A. J. Mallett O1/28/1966 K-25 [None] Transport safety analysis good N Engineering design for a to be co-constructed with the conceptual Design Report for Critical Assembly Division) O6/15/1966 K-25 [None] Engineering design for a to be co-constructed with the conceptual Design Report for Critical Assembly Bay ELA Conceptual Design Report for Critical Assembly Staff (of the Engineering Division) O7/29/1966 K-25 [None] Engineering design for a to be co-constructed with the conceptual Design Report for Critical Assembly Bay ELA		V D 1011		n. L. Mackini, A. J. Mailet		N-2J	02/01/1300	equipment/process design	1011	14	innits for in-plant OF <sub>6</sub> cylinders.
Rail Shipment of 10-Ton Cylinders Containing 2.1% Enriched UF <sub>6</sub> A. J. Mallett  O1/28/1966  K-25  [None]  transport safety analysis good N  Engineering design for a to be co-constructed will be co-construc			1								
Rail Shipment of 10-Ton Cylinders Containing 2.1% Enriched UF <sub>6</sub> A. J. Mallett 01/28/1966 K-25 [None] transport safety analysis good N  Engineering design for a to be co-constructed wit to be co-constructed wit to be co-constructed wit to be co-constructed with the containing and the containing an			1						memo, original,		
281 K-D-1893 2.1% Enriched UF <sub>6</sub> A. J. Mallett 01/28/1966 K-25 [None] transport safety analysis good N  Conceptual Design Report for Critical Assembly Bay ELA  Conceptual Design Report for Critical Assembly Division)  Conceptual Design Report for Critical Assembly Bay ELA  Conceptual Design Report for Critical Assembly Division)  O7/29/1966 K-25 [None] experiment plan/design good N  Authorizes off-site Shipr UF <sub>6</sub> cylinders with up to enrichments. Includes descriptions of experiments. Includes descriptions of experiments. Understand the Oak Richard Conceptual Design Criteria Change: Protective Structural Package  Criteria Change: Protective Structural Package  A. D. Callihan, A. P. Huber,	280	K-D-1858		A. J. Mallett	06/23/1965	K-25	[None]	transport safety analysis	good	N	
Conceptual Design Report for Critical Assembly Bay ELA Conceptual Design Report for Critical Assembly Bay ELA Conceptual Design Report for Critical Assembly Division)  Conceptual Design Report for Critical Assembly Bay ELA Division)  O7/29/1966  K-25 [None] Engineering design for a to be co-constructed with the conceptual point of the Engineering pood N Electron Linear Accelerate the Staff (of the Engineering pood N  Authorizes off-site Shipr UF <sub>6</sub> cylinders with up to enrichments. Includes process of experiment plan/design Criteria Change: Protective Structural Package A. D. Callihan, A. P. Huber,  Criteria Change: Protective Structural Package A. D. Callihan, A. P. Huber,			Rail Shipment of 10-Ton Cylinders Containing						memo, original,		
Conceptual Design Report for Critical Assembly Bay ELA Conceptual Design Report for Critical Assembly Bay ELA Conceptual Design Report for Critical Assembly Division)  Conceptual Design Report for Critical Assembly Bay ELA Division)  O7/29/1966  K-25 [None] Engineering design for a to be co-constructed with the conceptual point of the Engineering pood N Electron Linear Accelerate the Staff (of the Engineering pood N  Authorizes off-site Shipr UF <sub>6</sub> cylinders with up to enrichments. Includes process of experiment plan/design Criteria Change: Protective Structural Package A. D. Callihan, A. P. Huber,  Criteria Change: Protective Structural Package A. D. Callihan, A. P. Huber,	281	K-D-1893	2.1% Enriched UF <sub>6</sub>	A. J. Mallett	01/28/1966	K-25	[None]		1	N	
Conceptual Design Report for Critical Assembly Bay ELA  K-D-1912  K-D-1912 Revised  Conceptual Design Report for Critical Assembly Bay ELA  Conceptual Design Report for Critical Assembly Division)  Conceptual Design Report for Critical Assembly Bay ELA  Conceptual Design Report for Critical Assembly Division)  Conceptual Design Report for Critical Assembly Division  Conceptual Design Report for Critical Assembly Division  Conceptual Design Report for Critical Assembly Division  N Electron Linear Accelerate Property or Capture Property Division  N Authorizes off-site Shipp UF6 (Vinders with up to enrichments. Includes property Division Office Pro					, -,	<del></del>		.,,	<del></del>		<del>                                     </del>
Conceptual Design Report for Critical Assembly Bay ELA  Conceptual Design Report for Critical Assembly Bay ELA  Conceptual Design Report for Critical Assembly Division)  Conceptual Design Report for Critical Assembly Bay ELA  Conceptual Design Report for Critical Assembly Division)  Criteria Change: Protective Structural Package  Conceptual Design Report for Critical Assembly Division)  Staff (of the Engineering Division)  O7/29/1966  K-25 [None]  Experiment plan/design good  N  Authorizes off-site shipr  UF <sub>6</sub> cylinders with up to enrichments. Includes protective Structural Package  Criteria Change: Protective Structural Package  A. D. Callihan, A. P. Huber,											Engineering design for a selection to
282 K-D-1912 Bay ELA Division) 06/15/1966 K-25 [None] experiment plan/design good N Electron Linear Accelerate Conceptual Design Report for Critical Assembly Bay ELA Division) 07/29/1966 K-25 [None] experiment plan/design good N Authorizes off-site shipp UF <sub>6</sub> cylinders with up to enrichments. Includes good experiments of experiment plan/design good N Criteria Change: Protective Structural Package A. D. Callihan, A. P. Huber, Criteria Change: Protective Structural Package A. D. Callihan, A. P. Huber, Criteria Change: Protective Structural Package A. D. Callihan, A. P. Huber, Criteria Change: Protective Structural Package A. D. Callihan, A. P. Huber, Criteria Change: Protective Structural Package A. D. Callihan, A. P. Huber, Criteria Change: Protective Structural Package A. D. Callihan, A. P. Huber, Criteria Change: Protective Structural Package A. D. Callihan, A. P. Huber, Criteria Change: Protective Structural Package A. D. Callihan, A. P. Huber, Criteria Change: Protective Structural Package A. D. Callihan, A. P. Huber, Criteria Change: Protective Structural Package A. D. Callihan, A. P. Huber, Criteria Change: Protective Structural Package A. D. Callihan, A. P. Huber, Criteria Change: Protective Structural Package A. D. Callihan, A. P. Huber, Criteria Change: Protective Structural Package A. D. Callihan, A. P. Huber, Criteria Change: Protective Structural Package A. D. Callihan, A. P. Huber, Criteria Change: Protective Structural Package A. D. Callihan, A. P. Huber, Criteria Change: Protective Structural Package A. D. Callihan, A. P. Huber, Criteria Change: Protective Structural Package A. D. Callihan, A. P. Huber, Criteria Change: Protective Structural Package A. D. Callihan, A. P. Huber, Criteria Change: Protective Structural Package A. D. Callihan, A. P. Huber, Criteria Change: Protective Structural Package A. D. Callihan, A. P. Huber, Criteria Change: Protective Structural Package A. D. Callihan, A. P. Huber, Criteria Change: Protective Structural Package A. D. Callihan, A. P. Huber, Criteria Change: Pro				c. (( ( ( ) - ( ) - ( )							Engineering design for a critical mass lab
Conceptual Design Report for Critical Assembly Bay ELA  Conceptual Design Report for Critical Assembly Bay ELA  Staff (of the Engineering Division)  O7/29/1966  K-25  [None]  experiment plan/design  report, original, good  Authorizes off-site shipr  UF <sub>6</sub> cylinders with up to enrichments. Includes prescribing of experiment cylinders under water staff of experiment cylinders under water staff of experiment plan/design  Criteria Change: Protective Structural Package  A. D. Callihan, A. P. Huber,  Transport safety analysis, memo, original, performed at the Oak Ri			1						1		to be co-constructed with the Oak Ridge
283 K-D-1912 Revised Bay ELA Division) 07/29/1966 K-25 [None] experiment plan/design good N  Authorizes off-site shipt UF <sub>6</sub> cylinders with up to enrichments. Includes p descriptions of experiment cylinders under water su performed at the Oak Ri	282	K-D-1912	· · · · · · · · · · · · · · · · · · ·	·	06/15/1966	K-25	[None]	experiment plan/design		N	Electron Linear Acceleration (ORELA).
Authorizes offi-site shipr  UF <sub>6</sub> cylinders with up to enrichments. Includes p descriptions of experim cylinders under water si Criteria Change: Protective Structural Package A. D. Callihan, A. P. Huber, transport safety analysis, memo, original, performed at the Oak Ri			Conceptual Design Report for Critical Assembly	Staff (of the Engineering					report, original,		
UF <sub>6</sub> cylinders with up to enrichments. Includes productive Structural Package  A. D. Callihan, A. P. Huber,  Criteria Change: Protective Structural Package  A. D. Callihan, A. P. Huber,  Transport safety analysis,  memo, original,  performed at the Oak Ri	283	K-D-1912 Revised	Bay ELA	Division)	07/29/1966	K-25	[None]	experiment plan/design	good	N	
enrichments. Includes p descriptions of experim cylinders under water su Criteria Change: Protective Structural Package A. D. Callihan, A. P. Huber, transport safety analysis, memo, original, performed at the Oak Ri											Authorizes off-site shipment of 30-inch
enrichments. Includes p descriptions of experim cylinders under water su Criteria Change: Protective Structural Package A. D. Callihan, A. P. Huber, transport safety analysis, memo, original, performed at the Oak Ri									1		UF <sub>6</sub> cylinders with up to 4.5%
descriptions of experiment of the Control of the Co											enrichments. Includes photos and
Criteria Change: Protective Structural Package A. D. Callihan, A. P. Huber, transport safety analysis, memo, original, performed at the Oak Ri											·
Criteria Change: Protective Structural Package A. D. Callihan, A. P. Huber, transport safety analysis, memo, original, performed at the Oak Ri											descriptions of experiments involving 7
											cylinders under water submersion,
284 K-D-1918   for 30-in, diam UF <sub>c</sub> Cylinder   R. L. Macklin, A. J. Mallet   07/22/1966   K-25   [None]   operational/test/material data   good   Y   Fxperiments Facility.			Criteria Change: Protective Structural Package	A. D. Callihan, A. P. Huber,							performed at the Oak Ridge Critical
to the second se	284	K-D-1918	for 30-in. diam UF <sub>6</sub> Cylinder	R. L. Macklin, A. J. Mallet	07/22/1966	K-25	[None]	operational/test/material data	good	Υ	Experiments Facility.
Use of Nonapproved Equipment for UF6 Transfer			Use of Nonapproved Equipment for UF6 Transfer								
			1						memo, original		Unauthorized installation and use of a
	285	K-D-1925	1 3	A. J. Mallett	08/25/1966	K-25	[None]		1 1	N	UF <sub>6</sub> gas transfer line.

		Water Immersion Test of UF <sub>6</sub> Cylinders with						report, copy,		A good-quality original-issue report is
286	K-D-1987	Simulated Damage	A. J. Mallett	11/07/1967	K-25	[None]	operational/test/material data	1	Υ	present in the H. R. Dyer collection.
					Institute for Neutron		,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,	i		,
					Physics and					
					Technology,					
					Kernforschungszentru	1		1		
		Theory of Delayed Supercritical Excursions to			m, Karlsruhe, West			report, copy,		
287	KFK-153	Determine Coefficients of Fast Reactors	K. Ott	06/1963	Germany	[None]	experiment safety analysis	fair	N	
		Safe Separation Distances for BONUS Fuel					computational method/data	report, original,		
288	K-L-3082	Assemblies in Water	C. E. Newlon	10/30/1968	K-25	[None]	(2)	good	N	
						Not to be released to				
						the public or given				
		ORGDP Fuel Reprocessing Studies Progress				external distribution		report, copy,		
289	K-L-1780, Part 9	Report for Period Ending June 30, 1965	S. H. Smiley	07/28/1965	K-25	without approval	equipment/process design	good	N	
										Addresses use of overpack containers for
		New End-Loading Shipping Container for					computational method/data	report, copy,		30-inch UF <sub>6</sub> cylinders, for shipment of
290	K-L-6210	Unirradiated Fuel Assemblies	A. J. Mallett, C. E. Newlon	10/1968	K-25	[None]	(2), transport safety analysis	good	N	BONUS fuel assemblies.
		Nuclear Safety Analysis of 48- and 30-Inch Heavy-	,	,		, ,	computational method/data	report, original,		
291	K-L-6243	Wall UF <sub>6</sub> Cylinders	C. E. Newlon, R. G. Taylor	10/03/1969	K-25	[None]	(2), transport safety analysis	good	N	
		1				·	computational method/data	report, original,		
292	K-L-6249	Fissile Class I Shipment of UF <sub>6</sub> "Heel" Cylinders	C. E. Newlon, R. G. Taylor	12/19/1969	K-25	[None]	(2), transport safety analysis	good	N	
	-	Activity Determination for Uranium Hexafluoride					,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,	report, original,		
293	K-L-6252	Shipping Containers	C. E. Newlon, R. G. Taylor	03/10/1970	K-25	[None]	transport safety analysis	good	N	
233	K-L-0232	Nuclear Safety Analysis of Moderation-	C. L. Newlon, N. G. Taylor	03/10/1370	K-25	[INOTIE]	computational method/data	report, original,		
294	K-L-6255	Controlled 48-InID Heavy-Wall UF <sub>6</sub> Cylinders	C. E. Newlon, R. G. Taylor	08/10/1970	K-25	[None]	(2), transport safety analysis	good	N	
234	K-L-0233	Controlled 48-IIIID Heavy-wall OF <sub>6</sub> Cylinders	C. E. Newion, R. G. Taylor	08/10/19/0	K-23	[None]	(2), transport safety analysis	good	IN	Evoluing technical issues, manitoring
		Advantaging for House to a decrease details and in								Explains technical issues, monitoring
205	K I C21C	Monitoring for Uranium Accumulations in	D. C. Taulan	05 /24 /4072	к эг	[N 1		report, original,		equipment, and practices/program to
295	K-L-6316	Diffusion Plant Equipment	R. G. Taylor	05/21/1973	K-25	[None]		good report, original,	N	detect uranium accumulations.
200	KOA 704	Avenue Celial Avenue for Colonia	D. M. Dunton	04/25/4064	к эг	[N 1	computational method/data	1		
296	KOA-794	Average Solid Angles for Spheres The Effect of Density on the Safe U-235	D. W. Burton	04/25/1961	K-25	[None]	(1)	good	N	
207	KD 405		C E Namian	04/44/4063	к эг	[N 1	computational method/data	report, original,		Cinciles to K 1550
297	KR-185	Enrichment Criterion	C. E. Newlon A. D. Callihan, H. F. Henry,	04/11/1962	K-25	[None] Secret, declassified	(1) computational method/data	good	N	Similar to K-1550.
200	WC 220	Notes of Collins of Control II 225 Slob		00/20/4054	к эг			report, original,		
298	KS-230	Notes on Criticality of a Gaseous U-235 Slab	R. L. Macklin	09/28/1951	K-25	07/29/1960	(1)	good	N	
			A.D. Colliban II.E. Hanne			Corret declaration	computational mathed/data	ranart ariginal		
200	WC 200	C-f- Bin- Bin-main	A. D. Callihan, H. F. Henry,	42/20/4054	к эг	Secret, declassified	computational method/data	report, original,		
299	KS-260	Safe Pipe Dimensions	R. L. Macklin	12/20/1951	K-25	09/02/1959	(1), equipment/process design	good	N	
			A D Calliban II E Hanni			Course declaration	computational mathed/data	ranart ariginal		
200	WC 245		A. D. Callihan, H. F. Henry,	00/22/4052	W 25	Secret, declassified	computational method/data	report, original,		
300	KS-315	U-235 Critical Mass Dependence on Moderation	R. L. Macklin	09/22/1952	K-25	08/08/1960	(1), equipment/process design	good	N	
		The Internation of Bone Contained of Contained				Count de de catére d				
204	WC 247	The Interaction of Bare Systems of Containers:		00/25/4052	W 25	Secret, declassified	computational method/data	report, original,		
301	KS-317	Part One	L. Geller	09/26/1952	K-25	11/15/1960	(1), equipment/process design	good	N	
		The lateractics of Born Contains of Contains				Count de de catére d				
202	VC 247	The Interaction of Bare Systems of Containers:	I. Callan	42/46/4052	к эг	Secret, declassified	computational method/data	report, original,		
302	KS-347	Part Two	L. Geller	12/16/1952	K-25	06/30/1960	(1), equipment/process design	good	N	
202	VC 274	Analysis of a Descible Dediction Descript	L Callan II E IIanni	00/02/4052	к эг	Secret, declassified		report, original,		
303	KS-374	Analysis of a Possible Radiation Burst	L. Geller, H. F. Henry	06/02/1953	K-25	08/08/1960	criticality accident	good	N	
		Variation of Critical Parameters Between 11 225				Count dod:f:	computational			
20.	WC 200	Variation of Critical Parameters Between U-235	II 5 Hann C 5 **	40/22/4055	V 25	Secret, declassified	computational method/data	report, original,		
304	KS-399	Assays of 4.9 Percent and 93.4 Percent	H. F. Henry, C. E. Newlon	10/23/1953	K-25	08/08/1960	(1), equipment/process design	good	N	
		An Interestion Theory and the Analization :					computational	ranast saistas		
22-	WGA 50	An Interaction Theory and Its Application to	0.5.11	00/00/		fa. 1		report, original,		
305	KSA-58	Criticality Problems	C. E. Newlon	08/03/1956	K-25	[None]	(1), equipment/process design		N	
2	WGA 70			04/07/:			,	report, original,		
306	KSA-70	Use of Water for ORGDP Cascade Fire Control	H. F. Henry	01/07/1957	K-25	Unclassified	equipment/process design	good	N	
2	WGA 420	Safe Parameters for U-235 Assays Below Five Per		04/44/:		fa. 1		report, original,		
307	KSA-128	Cent	C. E. Newlon	04/11/1958	K-25	[None]	equipment/process design	good	N	
2	WGA 225	Analysis of Area Dosimetry Data Obtained from		04/05/:		fa. 1	ne 19 - 11 -	report, original,		
308	KSA-236	Criticality Tests	H. F. Henry, J. R. Knight	04/06/1960	K-25	[None]	criticality accident	good	N	
2	WG4 2266	Analysis of Area Dosimetry Data Obtained from		00/40/		fa. 1	100	report, original,		
309	KSA-236 Supplement 1	Criticality Tests	H. F. Henry, J. R. Knight	08/18/1960	K-25	[None]	criticality accident	good	N	

			T	Т						T
		Safe Concentration of Beryllium in Thin Alloy					computational method/data	report, original,		
310	K-TL-169	Reflectors	C. E. Newlon	05/25/1971	K-25	[None]	(1), equipment/process design	1	N	
310	K-1L-103	henectors	C. L. NEWION	03/23/13/1	K-25	[None]	(1), equipment/process design	good	IN.	+
							computational method/data	report, original,		
311	K-TL-232	30-Inch UF <sub>6</sub> Cold Traps	C. E. Newlon	04/04/1972	K-25	[None]	(2), equipment/process design	good	N	
		Nuclear Safety Analysis of the "Paducah Tiger"		- ,, - ,,		[]	(-),, p	report, original,		
312	K-TL-236	(Shipping Package for the 10-Ton UF <sub>6</sub> Cylinder)	C. E. Newlon	05/24/1972	K-25	[None]	transport safety analysis	good	N	
312	K-1L-250	(Shipping Fackage for the 10-1011 OF <sub>6</sub> Cylinder)	C. L. IVEWIOTI	03/24/13/2	K-25	[None]	transport safety analysis	good		
		Comparison of Congrete and Water Reflected					samputational mathed/data	roport original		
242	V TI 272	Comparison of Concrete- and Water-Reflected	C E Namian	11/10/1072	K 25	[N 1	computational method/data	report, original,		
313	K-TL-273	Uranium Hexafluoride Systems  A Generalized and Consistent Method for	C. E. Newlon	11/10/1972	K-25	[None]	(2), equipment/process design	good	N	
							computational mathed/data	roport original		
314	KY-294	Calculating the Critical Mass of Homogeneous Aqueous Uranyl Fluoride Solutions	O. W. Hermann	06/08/1959	Paducah	[None]	computational method/data	report, original,	N	
314	K1-294	Aqueous oranyi Fluoride Solutions	W. R. Pedigo, V. A. Smith,	06/08/1959	Paducan	[None]	(1)	good	IN	
		Testing of Ten Ten Uranium Havefluorida	1					roport original		
315	KY-500	Testing of Ten-Ton Uranium Hexafluoride	S. Bernstein, W. J. Hamer,	10/22/1965	Daducah	[Nono]	transport safety analysis	report, original,	N	
313	K1-500	Cylinders	J. L Myers W. P. Rodigo	10/22/1905	Paducah	[None]	transport safety analysis	good	IN	
		Tosting of Ton Ton Canacity Uranium	J. L Myers, W. R. Pedigo,					roport original		
216	KV D 2022	Testing of Ten-Ton Capacity Uranium	W. J. Hamer, V. A. Smith,	12/07/1064	Dadusah	[None]	transport safety analysis	report, original,	N	
316	KY-D-2032	Hexafluoride Shipping Cylinders	S. Bernstein	12/07/1964	Paducah	[None]	transport safety analysis	good	N	
			C. P Baker, H. K. Daglian,					roport possti		
			G. Friedlander, M. G.			Course declaration		report, negative photostatic		A honohmark model is provided in
247	14.424	Matau Bailau	Holloway, D. W. Kerst, R.	00/04/4044	1.0011	Secret, declassified		I I	.,	A benchmark model is provided in
317	LA-134	Water Boiler	E. Schreiber	09/04/1944	LANL	05/01/1973	experimental criticality data	copy, fair	Υ	IHECSBE report IEU-SOL-THERM-004.  Possibly the first critical experiments
										using Pu solution. Includes letter from K-
										25 to Los ALamo (05/22/1949)
										requesting certain dimensional details
						6				not contained in the report, plus a
						Secret, declassified		report, original,		handwritten page (no date) containing
318	LA-272	Critical Mass of a Water-Tamped 49 Solution	B. T. Feld, L. Slotin	05/14/1945	LANL	11/04/1955	experimental criticality data	fair	Υ	the requested information.
		Critical Masses of Enriched Uranium Hydrides	C. P. Baker, M. G.			Secret, declassified		report, original,		
319	LA-618	and Some Related measurement	Holloway	02/03/1947	LANL	05/01/1973	experimental criticality data	fair	Y	Original photos included.
										Contains interesting assessment as to
		Radiation Doses in the Pajarito Accident of May				Secret, declassified	experimental criticality data,	report, original,		the possible causes of the observed blue
320	LA-687	21, 1946	J. G. Hoffman	05/26/1948	LANL	12/21/1951	dosimetry	good	Υ	glow.
		Polythene - 25 Critical Assembly and Neutron	H. C. Paxton, G. A.			Secret, declassified		report, original,		
321	LA-749	Distribution Studies	Linenburger	09/30/1949	LANL	05/01/1973	experimental criticality data	fair	Y	Original photos included.
			V. Josephson, R. W. Paine	/ /		Secret, declassified		report, original,		
322	LA-1155	Oralloy Shape Factor Measurements	Jr., L. L. Woodward	08/08/1950	LANL	02/19/1958	experimental criticality data	good	Υ	
						Secret, declassified		report, original,		
323	LA-1209	Measurements on Untamped Oralloy Assembly	J. D. Orndoff, H. C. Paxton	02/08/1951	LANL	04/01/1964	experimental criticality data	good	Υ	
		Critical Masses of Oralloy at Reduced	J. D. Orndoff, H. C. Paxton,			Secret, declassified		report, copy,		
324	LA-1251	Concentrations and Densities	G. E. Hansen	05/01/1951	LANL	02/28/1958	experimental criticality data	poor	Υ	
										The accident of 02/01/1951 involving the
						Secret, declassified		report, original,		"Aquarium" assembly device;
325	LA-1289	A Study of an Accidental Radiation Burst	R. W. Paine Jr.	03/20/1951	LANL	02/10/1958	experimental criticality data	good	Υ	investigation of cause.
						Secret, declassification				
		Burst Characteristics Associated with the Slow				indicated but no date		report, original,		
326	LA-1441	Assembly of Fissionable Materials	G. E. Hansen	07/1952	LANL	provided	criticality accident	good	N	
1			E. C. Mallary, G. E. Hansen,	1						The accident of 04/18/1952 involving the
1		Neutron Burst from a Cylindrical Untamped	G. A. Linenberger, D. P.			Secret, declassified		report, original,		"Jemima" assembly device; investigation
327	LA-1477	Oralloy Assembly	Wood	07/22/1952	LANL	02/10/1958	experimental criticality data	good	Y	of cause.
		Neutron Distribution Measurements at Pajarito		1 T		Secret, declassified		report, original,		
328	LA-1478	by Means of Photographic Emulsions	D. S. Young	05/15/1952	LANL	02/10/1958	nuclear measurement/data	good	N	
		Practical Aspects of Pajarito Neutron				Secret, declassified		report, original,		
329	LA-1604	Multiplication Measurements	F. F. Hart, E. C. Mallary	09/1953	LANL	03/11/1958	nuclear measurement/data	good	N	
		Lady Godiva: An Unreflected Uranium-235				Secret, declassified		report, original,		A benchmark model is provided in
330	LA-1614	Critical Assembly	R. E. Peterson	09/1953	LANL	02/06/1958	experimental criticality data	good	Υ	IHECSBE report HEU-MET-FAST-001.
		Neutron Detector Traverses in the Topsy and	G. A. Linenberger, L. L.			Secret, declassified		report, original,		
331	LA-1653	Godiva Critical Assemblies	Lowry	06/1953	LANL	08/31/1957	nuclear measurement/data	good	N	

	T			T				1		Selected benchmark models are
		Bare Critical Assemblies of Oralloy at				Secret, declassified		report, original,		provided in IHECSBE reports IEU-MET-
332	LA-1671	Intermediate Concentrations of U-235	H. C. Paxton	05/1964	LANL	03/16/1960	experimental criticality data	good	Υ	FAST-001 and -002.
332	LA-10/1	Material Replacement Measurements in Topsy	L. B. Engle, G. E. Hansen,	03/1904	LAINL	Secret, declassified	experimental criticality data	report, original,	<del>- '</del>	FA31-001 and -002.
222	LA-1708	- II		07/1954	LANL	02/10/1958	over a rim antal ariticality data	1	Υ	
333	LA-1708	and Godiva Assemblies	H. C. Paxton	07/1954	LAINL		experimental criticality data	good		
224	14 4722	Critical Masses of Graphite-Tamped	I C IIt	05/1051		Secret, declassified		report, original,		
334	LA-1732	Heterogeneous Oy-Graphite Systems	J. C. Hoogterp	05/1954	LANL	02/06/1958	experimental criticality data	good	Y	
									1	
		5				Secret, declassification			1	
		Emission Probabilities of Prompt Neutrons from				indicated but no date		report, original,		
335	LA-1863	Spontaneous and Neutron-Induced Fission	R. B. Leachman	12/1954	LANL	provided	nuclear measurements/data	good	N	
		Calculations of the Critical Mass of UF <sub>6</sub> as a				Secret, declassified	computational method/data	report, original,	1	
336	LA-1874	Gaseous Core, With Reflectors of D₂O, Be and C	G. I. Bell	02/1955	LANL	02/06/1958	(1)	good	N	
										Folder also contains an original copy of
									1	the report in unclassified form, report
			E. C. Mallary, H. C. Paxton,			Secret, declassified		report, original,	1	dated 02/1955 and numbered as "LA-
337	LA-1875	Safety Tests for the Storage of Fissile Units	R. H. White	02/1955	LANL	05/01/1973	experimental criticality data	good	Υ	1875 (Deleted)".
		, ,		,						Folder also contains an original copy of
									1	the report classified as "confidential",
									1	report dated 04/1956 and numbered as
		Critical Masses of Fissionable Materials as Basic				Secret, declassified		report, original,	1	"LA-1958 (Deleted)", declassified on
338	LA-1958	Nuclear Safety Data	H. C. Paxton	01/1955	LANL	02/10/1958	handbook	good	N	01/31/1957.
	EA 1930	Fission Neutron Spectrum of U <sup>235</sup> from 0.2 to 3	TI. C. I daton	01/1333		02/10/1550	Hallabook	-	<del></del>	01/31/1337.
220		•		05/4055		fa. 1		report, original,		
339	LA-1916	MeV	L. Cranberg, N. G. Nereson	05/1955	LANL	[None]	nuclear measurements/data	good	N	
								report, original,		
340	LA-1938	Paraffin Cylinders to Measure Neutron Energies	D. S. Young	07/1955	LANL	[None]	nuclear measurements/data	good	N	
						Confidential,			1	
						declassification			1	
		Preliminary Survey of Uranium Metal				indicated but no date		report, original,	1	
341	LA-2023	Exponential Columns	J. J. Neuer, C. B. Stewart	01/1956	LANL	provided	experimental criticality data	good	N	
						Confidential,			1	
		Critical Masses of Oralloy Lattices Immersed in				declassified		report, original,	1	
342	LA-2026	Water	J. C. Hoogterp	11/1955	LANL	08/04/1960	experimental criticality data	good	Y	
										Folder contains an indication the report
343	LA-2029	REPORT IS MISSING FROM FOLDER							1	was loaned to J. T. Mihalczo.
						Secret, declassification			1	
			G. A. Jarvis, G. A.			indicated but no date		report, original,	1	
344	LA-2044	Plutonium-Metal Critical Assemblies	Linenberger, H. C. Paxton	05/1956	LANL	provided	experimental criticality data	good	Y	
										The report copy is extensively marked up
									1	and otherwise altered; apparently
									1	Callihan used this copy as a working copy
									1	for development of TID-7016. Other
			A. D. Callihan, W. J.			Confidential,			1	draft materials for TID-7016 and
			Ozeroff, H. C. Paxton, C. L.			declassified		report, original,	1	correspondence are included in the
345	LA-2063	Nuclear Safety Guide	Schuske	08/1956	LANL	02/03/1958	handbook	poor	N	folder.
5 +5	2000			55, 2550		Confidential,		F 50.	<u>;</u>	1
		Critical Assembly of Uranium Metal at an				declassified		report, original,	í	A benchmark model is provided in
346	LA-2085	Average U <sup>235</sup> Concentration of 16-1/4%	J. J. Neuer	10/1956	LANL	03/16/1960	experimental criticality data	1	Y	IHECSBE report IEU-MET-FAST-002.
340	LA-2003	Average U Concentration of 10-1/476	G. E. Hansen, J. C.	10/1330	LAINL	03/10/1300	experimental criticality data	good	<del> </del>	See IHECSBE report HEU-MET-THERM-
		Parullium Patlacted Granhita Madarata						roport origins!	í	
247	I A 2141	Beryllium-Reflected, Graphite-Moderated	Hoogterp, J. D. Orndoff, H.	07/1057	LAND	[None]	oungriss antal cutting literature	report, original,		002 (none of the experiments are judged
347	LA-2141	Critical Assemblies	C. Paxton	07/1957	LANL	[None]	experimental criticality data	good	Y	acceptable as benchmarks).
2.2		1, 2, 1, 1, 1, 2, 3	D 6 W	40/405=		fa. 3		report, original,	1	
348	LA-2158	(n,2n) Study of Be <sup>9</sup>	D. S. Young	10/1957	LANL	[None]	nuclear measurement/data	good	N	
								report, original,		
349	LA-2177	Neutron Scattering by U <sup>235</sup> , Pu <sup>239</sup> , and U <sup>238</sup>	L. Cranberg	01/1959	LANL	[None]	nuclear measurement/data	good	N	
		Correlations of Experimental and Theoretical					experimental criticality data,		1	
		Critical Data Comparative Reliability, Safety					computational methods/data	report, original,	í	
350	LAMS-2537	Factors for Criticality Control	H. C. Paxton	03/1961	LANL	[None]	(1)	good	Y	
		Numerical Formulation and Solution of Neutron					computational method/data	report, original,	1	
	1	Transport Problems	B. G. Carlson	11/11/1963	LANL	[None]	(2)	good	N	
351	LA-2996	Transport Problems	D. C. Carison	11/11/1505	LANC	[	\ <del>-</del> /			
351	LA-2996	A Method of Moments for Solving the Neutron	Di di dansan	11/11/1505	DAIVE	[rione]	computational method/data	report, original,		

353	LA-3219-MS	Critical Assemblies of Fissionable Materials	C. B. Mills	10/1959	LANL	[None]	experimental criticality data	report, original, good	N	Summary report of critical experiments.
							computational method/data	report, original,		
354	LA-3221-MS	Reactor Minimum Critical Dimensions	C. B. Mills	10/1959	LANL	[None]	(1)	good	N	
		Criticality Control in Operations with Fissile						report, original,		
355	LA-3366	Material	H. C. Paxton	12/1964	LANL	[None]	handbook	good	Y	
356	LA-3366 (Rev)	Criticality Control in Operations with Fissile Material	H. C. Paxton	11/1966	LANL	[None]	handbook	report, original, good	Υ	
	LA-5500 (NeV)	iviateriai	n. c. raxton	11/1900	LAINL	[None]	Hallubook	good		A low-power (12 kW) reactor
		Hydro A Small, Water-Cooled, and Water-						report, original,		purposefully designed to drive
357	LA-3374	Reflected Neutron Source	W. Bernard	12/1965	LANL	[None]	experimental criticality data	good	Υ	exponential columns.
		Neutron Cross Sections for <sup>235</sup> U and <sup>238</sup> U in the	JJ. H. Berlijn, R. E.					report, original,		
358	LA-3527	Energy Range 1 keV to 14 MeV	Hunter, C. C. Cremer	08/1968	LANL	[None]	nuclear measurement/data	good	N	
								report, original,		
359	LA-3611	A Review of Criticality Accidents	W. R. Stratton	01/1967	LANL	[None]	criticality accident	good	Υ	
		Development of Calculational Methods for Fast					computational method/data	report, original,		
360	LA-3683-MS	Reactor Safety Analysis at LASL	C. A. Anderson Jr.	04/24/1967	LANL	[None]	(2)	good	N	
										The document is a draft copy of the
		Data skipa Idantification and Analysis of								report (abstract through reference list);
		Detection, Identification and Analysis of Fissionable Isotopes Based on Differential Group						report, draft,		has a handwritten note "Later Issued as LA-3741 with minor revisions (editing
361	LA-3741		G. R. Keepin	01/1967	LANL	[None]		fair	N	only)".
301	LA-3741	Yields and Spectra of Delayed Fission Neutrons टातारका जातानाजां जा नाजालहुटाहरूक उन्नाहास्त्र	о. к. кееріп	01/1507	LANL	[None]		lan		Only).
		Containing 235U, 238U, and Carbon for Various					computational method/data	roport original		
362	LA-3883-MS	C/ <sup>235</sup> U Ratios and <sup>235</sup> U Enrichments	L. B. Engle, W. R. Stratton	12/15/1967	LANL	[None]	(2), equipment/process design	report, original,	N	
302	LA-3003-IVI3	Neutron Flux Measurements in Uranium Metal	L. B. Eligie, W. K. Stratton	12/13/1907	LAINL	[None]	(2), equipment/process design	report, original,	IN	
363	LA-3934	Exponential Columns of 6.53% and 9.12% <sup>235</sup> U	R. G. Steinke	01/1968	LANL	[None]	experimental criticality data	good	N	
303	LA-3334	Operating Procedures for the Pajarito Site	II. G. Stellike	01/1508	LANE	[INOTIE]	experimental criticality data	report, original,		
364	LA-4037-SOP, Rev.	Critical Assembly Machine	J. D. Orndoff, H. C. Paxton	01/1973	LANL	[None]	experiment plan/design	good	N	
			, , , , , , , , , , , , , , , , , , , ,					report, original,		
365	LA-4037-SOP, Rev., Suppl. 1	Pajarito Plan for Radiation Emergency	H. C. Paxton	05/1973	LANL	[None]	criticality accident	good	N	
		The Data of Nuclear Reactor Physics, 1967-1968:						report, original,		
366	LA-4225-MS	A Bibliography	J. Furnish	09/17/1969	LANL	[None]	handbook/bibliography	good	N	
		Safety Analysis for the Los Alamos Critical-	W. U. Geer, P. G. Koontz,					report, original,		
367	LA-4273	Assembly Facility	J. D. Orndoff, H. C. Paxton	05/06/1969	LANL	[None]	experiment safety analysis	good	N	
		Application of S <sub>n</sub> Calculations to the Evaluation								
		of a Shipping Container for Small Quantities of						report, original,		
368	LA-4325	Fissile Radioactive Material	D. R. Smith	10/24/1969	LANL	[None]	transport safety analysis	good	N	
369	LA-4484-MS	Twenty-Five Group Cross Sections Used in the	I Canin	07/1970	LANL	[None]	nuclear measurement (data	report, original,	N	
309	LA-4464-IVIS	Los Alamos Rover Program	J. Sapir	07/1970	LAINL	[None]	nuclear measurement/data	good report, original,	IN	
370	LA-4671	An Early History of Criticality Safety	R. Reider	05/1971	LANL	[None]	experimental criticality data	good	N	
3,0	51 1072	ran Edity History or enticonicy surecy	THE	05/15/1	2	[Hone]	experimental entireanty data	report, original,		
371	LA-4879	Synergy and Artificial Intelligence	D. R. Conant	03/1972	LANL	[None]		good	N	
								report, original,		
372	LA-5189-MS	Nuclear Furnace-1 Test Report	W. L. Kirk	03/1973	LANL	[None]		good	N	
							computational method/data	report, original,		
373	LAMS-2240	Reactor Computing Practices	C. B. Mills	06/1958	LANL	[None]	(1)	good	N	
		Neutron Cross Sections for Fast and	0.0.453	40/4055		fa. 1		report, original,		
374	LAMS-2255	Intermediate Nuclear Reactors	C. B. Mills	10/1958	LANL	[None]	nuclear measurement/data	good	N	ļ
							nuclear measurement/data, computational method/data	report, original,		
375	LAMS-2288	Physics of Intermediate Reactors	C. B. Mills	01/1959	LANL	[None]	(1)	good	N	
3/3	213 2200	, s.es of intermediate fleations	C. D. Willia	01/1555	LOUNE	inone	nuclear measurement/data,	P200	N	
							computational method/data	report, original,		
376	LAMS-2288 (Suppl. 1)	Physics of Intermediate Reactors	C. B. Mills	04/1959	LANL	[None]	(1)	good	N	
								report, original,		
377	LAMS-2415	Critical Data for Nuclear Safety Guidance	H. C. Paxton	02/1960	LANL	[None]	handbook/bibliography	good	N	
		Critical Masses of Composites of Oy and Pu-239-	D. M. Barton, W. Bernard,					report, original,		
378	LAMS-2489	240 in Flattop Geometry	G. E. Hansen	12/1960	LANL	[None]	experimental criticality data	good	Υ	
		Six and Sixteen Group Cross Sections for Fast						report, original,		
379	LAMS-2543	and Intermediate Critical Assemblies	G. E. Hansen, W. H. Roach	11/1961	LANL	[None]	nuclear measurement/data	good	Υ	

			T				1			
		Critical Mass Measurements of Oy and Pu Cores		I				report, original,		
380	LAMS-2579	in Spherical Aluminum Reflectors	D. P. Wood, B. Pena	06/1961	LANL	[None]	experimental criticality data	good	Y	
		Hazards Evaluation for the Los Alamos Critical	W. U. Geer, P. G. Koontz,	I				report, original,		
381	LAMS-2698 Revised	Assembly Facility	J. D. Orndoff, H. C. Paxton	04/1962	LANL	[None]	experiment safety analysis	good	N	Replaced by LA-4273
	+		G. E. Hansen, D. P. Wood,					report, original,		
202	14446 2744			00/1000	1.441	[N 1		1		
382	LAMS-2744	Large Diameter U(93.2%) Slabs	B. Pena	06/1962	LANL	[None]	experimental criticality data	good	Υ	
				I						
		Anisotropic Scattering in the Transport Equation	[ ]	I			computational method/data	report, original,		
383	LAMS-2873	An Evaluation of Common Approximations	K. D. Lathrop	03/01/1963	LANL	[None]	(1)	good	N	
		Study of Fission Neutron Spectra with High-			<u> </u>			report, original,		
384	LAMS-2883	Energy Activation Detectors	J. A. Grundl	03/1963	LANL	[None]	nuclear measurement/data	good	N	
364	LAIVI3-2003	Energy Activation Detectors	J. A. Grundi	03/1903	LAINL	[NOTIE]	nuclear measurement/uata	<u> </u>	IN	
				I				report, original,		
385	LAMS-3067	Los Alamos Critical-Mass Data	H. C. Paxton	04/1964	LANL	[None]	handbook/bibliography	good	Y	
		A Neutron Detector Having Uniform Sensitivity	A. O. Hanson, J. L.	1				report, original,		
386	MDDC-972 (LADC-409)	from 10 keV to 3 MeV	McKibben	02/11/1947	LANL	[None]	nuclear measurement/data	good	N	
	+		R. A. Wolfe, D. A. Edling,							
		1	D. F. Giessing, J. B. Kahle,	1				report, original,		Maximum neutron multiplication was ~
		1						1		
387	MLM-1395		W. F. Stubbins	01/09/1967	Mound Laboratory	[None]	experimental criticality data	good	Y	1.025.
		Subcritical Neutron Multiplication Experiment	[ ]	1						
		with Four SNAP-19B (IRHS) Heat Sources	[ ]	I				report, original,		Maximum neutron multiplication was ~
388	MLM-1523	Containing Plutonium-238	R. A. Wolfe, W. F. Stubbins	01/24/1969	Mound Laboratory	[None]	experimental criticality data	good	N	1.19.
300	1	Nuclear Safety of DOT Special Permit No. 6000		51,2.,1505	ouu zuborutory	į		10000	- "	
		1		I						
		Package for Large Quantities of Fissile Waste		I				report, original,		
389	MLM-1714	Material	R. A. Wolfe	02/20/1970	Mound Laboratory	[None]	transport safety analysis	good	N	
		Critical Consultation of Consultation of	[ ]	I						
		Critical Experiments on a Small Reactor of	[ ]	I						
		Enriched U-235 with an AL-H2O Moderator and	1	I				CSIRC-		
390	MonP-357	D2O, Be and H2O Reflectors	MM Mann, et al	8/18/1947	K-25	Unknown	Early Small Reactor Studies	Electronic	T-Vol 1A	
										Original photos included. Experiments
			[ ]	I						performed in what is now Building 3019
			[ ]	I						-
			[ ]	I						at the ORNL site. Enriched uranium (95%
			[ ]	I						<sup>235</sup> U enrichment) in D <sub>2</sub> O solution in
				I						
		Criticality Studies on Enriched Uranium Heavy		1		Secret, declassified		report, original,		cylinders, arranged in an lattice within a
391	MonP-454	Water Systems	A. H. Snell	12/15/1947	ORNL	02/10/1958	experimental criticality data	fair	Y	D <sub>2</sub> O-filled tank.
										Appears to be an informal (internal
				I						LANL) version of LA-3611, developed for
			[ ]	I				report, original,		A Company of the Comp
			1							a special American Nuclear Society
392	N-2-1713	1 .	l i					1		
		A Review of Criticality Accidents	W. R. Stratton	03/03/1960	LANL	[None]	criticality accident	good	N	session regarding nuclear safety.
		A Review of Criticality Accidents	W. R. Stratton	03/03/1960	LANL North American	[None]	criticality accident	1	N	
			W. R. Stratton  A. T. Biehl, E. R. Cohen, D.	03/03/1960			criticality accident	1	N	
393	NAA-SR-148	A Measurement of the Neutron Temperature	A. T. Biehl, E. R. Cohen, D.		North American Aviation, Inc., Downey	Secret, declassified		good report, original,		
393	NAA-SR-148	A Measurement of the Neutron Temperature		03/03/1960	North American Aviation, Inc., Downey CA		criticality accident nuclear measurement/data	good	N N	
393	NAA-SR-148	A Measurement of the Neutron Temperature Effect Using Europium Oxide Foils	A. T. Biehl, E. R. Cohen, D.		North American Aviation, Inc., Downey CA North American	Secret, declassified		good report, original, good		
		A Measurement of the Neutron Temperature Effect Using Europium Oxide Foils Correction Factors for Measurements with	A. T. Biehl, E. R. Cohen, D. Woods	09/25/1951	North American Aviation, Inc., Downey CA North American Aviation, Inc., Downey	Secret, declassified 03/24/1958	nuclear measurement/data	good report, original, good report, original,	N	
393 394	NAA-SR-148 NAA-SR-1076	A Measurement of the Neutron Temperature Effect Using Europium Oxide Foils Correction Factors for Measurements with	A. T. Biehl, E. R. Cohen, D.		North American Aviation, Inc., Downey CA North American	Secret, declassified		good report, original, good		
		A Measurement of the Neutron Temperature Effect Using Europium Oxide Foils Correction Factors for Measurements with	A. T. Biehl, E. R. Cohen, D. Woods	09/25/1951	North American Aviation, Inc., Downey CA North American Aviation, Inc., Downey	Secret, declassified 03/24/1958	nuclear measurement/data	good report, original, good report, original,	N	
		A Measurement of the Neutron Temperature Effect Using Europium Oxide Foils Correction Factors for Measurements with	A. T. Biehl, E. R. Cohen, D. Woods	09/25/1951	North American Aviation, Inc., Downey CA North American Aviation, Inc., Downey	Secret, declassified 03/24/1958	nuclear measurement/data	good report, original, good report, original,	N	
		A Measurement of the Neutron Temperature Effect Using Europium Oxide Foils Correction Factors for Measurements with	A. T. Biehl, E. R. Cohen, D. Woods	09/25/1951	North American Aviation, Inc., Downey CA North American Aviation, Inc., Downey CA	Secret, declassified 03/24/1958 [None] Unclassified,	nuclear measurement/data	good report, original, good report, original,	N	
		A Measurement of the Neutron Temperature Effect Using Europium Oxide Foils Correction Factors for Measurements with	A. T. Biehl, E. R. Cohen, D. Woods	09/25/1951	North American Aviation, Inc., Downey CA North American Aviation, Inc., Downey CA Atomics International,	Secret, declassified 03/24/1958  [None]  Unclassified, intended for internal	nuclear measurement/data	good report, original, good report, original,	N	
		A Measurement of the Neutron Temperature Effect Using Europium Oxide Foils  Correction Factors for Measurements with Cadmium Covered Foils	A. T. Biehl, E. R. Cohen, D. Woods	09/25/1951	North American Aviation, Inc., Downey CA North American Aviation, Inc., Downey CA  Atomics International, North American	Secret, declassified 03/24/1958  [None]  Unclassified, intended for internal use only, may not be	nuclear measurement/data nuclear measurement/data	good report, original, good report, original,	N	session regarding nuclear safety.
		A Measurement of the Neutron Temperature Effect Using Europium Oxide Foils Correction Factors for Measurements with	A. T. Biehl, E. R. Cohen, D. Woods	09/25/1951	North American Aviation, Inc., Downey CA North American Aviation, Inc., Downey CA  Atomics International, North American	Secret, declassified 03/24/1958  [None]  Unclassified, intended for internal	nuclear measurement/data	good report, original, good report, original,	N	
		A Measurement of the Neutron Temperature Effect Using Europium Oxide Foils  Correction Factors for Measurements with Cadmium Covered Foils  Criticality Criteria for Fabrication of 2% Enriched	A. T. Biehl, E. R. Cohen, D. Woods	09/25/1951	North American Aviation, Inc., Downey CA North American Aviation, Inc., Downey CA  Atomics International, North American	Secret, declassified 03/24/1958  [None]  Unclassified, intended for internal use only, may not be	nuclear measurement/data nuclear measurement/data	good report, original, good report, original, good	N	session regarding nuclear safety.
394	NAA-SR-1076	A Measurement of the Neutron Temperature Effect Using Europium Oxide Foils  Correction Factors for Measurements with Cadmium Covered Foils  Criticality Criteria for Fabrication of 2% Enriched	A. T. Biehl, E. R. Cohen, D. Woods D. H. Martin	09/25/1951	North American Aviation, Inc., Downey CA North American Aviation, Inc., Downey CA  Atomics International, North American Aviation, Inc., location	Secret, declassified 03/24/1958  [None]  Unclassified, intended for internal use only, may not be published without the	nuclear measurement/data nuclear measurement/data facility/process/storage	good report, original, good report, original, good report, copy,	N N	session regarding nuclear safety.  Determines safe limits for four different
394	NAA-SR-1076	A Measurement of the Neutron Temperature Effect Using Europium Oxide Foils  Correction Factors for Measurements with Cadmium Covered Foils  Criticality Criteria for Fabrication of 2% Enriched	A. T. Biehl, E. R. Cohen, D. Woods D. H. Martin	09/25/1951	North American Aviation, Inc., Downey CA North American Aviation, Inc., Downey CA  Atomics International, North American Aviation, Inc., location	Secret, declassified 03/24/1958  [None]  Unclassified, intended for internal use only, may not be published without the approval of AEC.	nuclear measurement/data nuclear measurement/data facility/process/storage	good report, original, good report, original, good report, copy,	N N	session regarding nuclear safety.  Determines safe limits for four different
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394	NAA-SR-1076	A Measurement of the Neutron Temperature Effect Using Europium Oxide Foils  Correction Factors for Measurements with Cadmium Covered Foils  Criticality Criteria for Fabrication of 2% Enriched	A. T. Biehl, E. R. Cohen, D. Woods D. H. Martin	09/25/1951	North American Aviation, Inc., Downey CA North American Aviation, Inc., Downey CA Atomics International, North American Aviation, Inc., location not specified  Atomics International, North American	Secret, declassified 03/24/1958  [None]  Unclassified, intended for internal use only, may not be published without the approval of AEC.  Unclassified, intended for internal	nuclear measurement/data nuclear measurement/data facility/process/storage analysis	good report, original, good report, original, good report, copy,	N N	session regarding nuclear safety.  Determines safe limits for four different steps in a manufacturing process.
394 395	NAA-SR-1076  NAA-SR-Memo-4099	A Measurement of the Neutron Temperature Effect Using Europium Oxide Foils  Correction Factors for Measurements with Cadmium Covered Foils  Criticality Criteria for Fabrication of 2% Enriched Uranium Hollow Slugs	A. T. Biehl, E. R. Cohen, D. Woods  D. H. Martin  T. S. Moy	09/25/1951 10/15/1954 07/08/1959	North American Aviation, Inc., Downey CA North American Aviation, Inc., Downey CA Atomics International, North American Aviation, Inc., location not specified Atomics International, North American Aviation, Inc., location	Secret, declassified 03/24/1958  [None]  Unclassified, intended for internal use only, may not be published without the approval of AEC.  Unclassified, intended for internal use only, may not be published without the	nuclear measurement/data nuclear measurement/data facility/process/storage analysis	good report, original, good report, original, good report, copy,	N N N T-CSIRC, Vol-	session regarding nuclear safety.  Determines safe limits for four different steps in a manufacturing process.
394 395	NAA-SR-1076	A Measurement of the Neutron Temperature Effect Using Europium Oxide Foils  Correction Factors for Measurements with Cadmium Covered Foils  Criticality Criteria for Fabrication of 2% Enriched Uranium Hollow Slugs	A. T. Biehl, E. R. Cohen, D. Woods D. H. Martin	09/25/1951	North American Aviation, Inc., Downey CA North American Aviation, Inc., Downey CA Atomics International, North American Aviation, Inc., location not specified Atomics International, North American Aviation, Inc., location	Secret, declassified 03/24/1958  [None]  Unclassified, intended for internal use only, may not be published without the approval of AEC.  Unclassified, intended for internal use only, may not be	nuclear measurement/data nuclear measurement/data facility/process/storage analysis	good report, original, good report, original, good report, copy,	N N	session regarding nuclear safety.  Determines safe limits for four different steps in a manufacturing process.
394 395	NAA-SR-1076  NAA-SR-Memo-4099	A Measurement of the Neutron Temperature Effect Using Europium Oxide Foils  Correction Factors for Measurements with Cadmium Covered Foils  Criticality Criteria for Fabrication of 2% Enriched Uranium Hollow Slugs	A. T. Biehl, E. R. Cohen, D. Woods  D. H. Martin  T. S. Moy	09/25/1951 10/15/1954 07/08/1959	North American Aviation, Inc., Downey CA North American Aviation, Inc., Downey CA Atomics International, North American Aviation, Inc., location not specified  Atomics International, North American Aviation, Inc., location of specified  Atomics International, North American Aviation, Inc., location not specified	Secret, declassified 03/24/1958  [None]  Unclassified, intended for internal use only, may not be published without the approval of AEC.  Unclassified, intended for internal use only, may not be published without the	nuclear measurement/data nuclear measurement/data facility/process/storage analysis	good report, original, good report, original, good report, copy,	N N N T-CSIRC, Vol-	session regarding nuclear safety.  Determines safe limits for four different steps in a manufacturing process.
394 395	NAA-SR-1076  NAA-SR-Memo-4099	A Measurement of the Neutron Temperature Effect Using Europium Oxide Foils  Correction Factors for Measurements with Cadmium Covered Foils  Criticality Criteria for Fabrication of 2% Enriched Uranium Hollow Slugs	A. T. Biehl, E. R. Cohen, D. Woods  D. H. Martin  T. S. Moy	09/25/1951 10/15/1954 07/08/1959	North American Aviation, Inc., Downey CA North American Aviation, Inc., Downey CA Atomics International, North American Aviation, Inc., location not specified  Atomics International, North American Aviation, Inc., location and specified  Atomics International, North American Aviation, Inc., location not specified	Secret, declassified 03/24/1958  [None]  Unclassified, intended for internal use only, may not be published without the approval of AEC.  Unclassified, intended for internal use only, may not be published without the	nuclear measurement/data nuclear measurement/data facility/process/storage analysis	good report, original, good report, original, good report, copy,	N N N T-CSIRC, Vol-	session regarding nuclear safety.  Determines safe limits for four different steps in a manufacturing process.
394 395	NAA-SR-1076  NAA-SR-Memo-4099	A Measurement of the Neutron Temperature Effect Using Europium Oxide Foils  Correction Factors for Measurements with Cadmium Covered Foils  Criticality Criteria for Fabrication of 2% Enriched Uranium Hollow Slugs	A. T. Biehl, E. R. Cohen, D. Woods  D. H. Martin  T. S. Moy	09/25/1951 10/15/1954 07/08/1959	North American Aviation, Inc., Downey CA North American Aviation, Inc., Downey CA Atomics International, North American Aviation, Inc., location not specified  Atomics International, North American Aviation, Inc., location of specified  Atomics International, North American Aviation, Inc., location not specified	Secret, declassified 03/24/1958  [None]  Unclassified, intended for internal use only, may not be published without the approval of AEC.  Unclassified, intended for internal use only, may not be published without the	nuclear measurement/data nuclear measurement/data facility/process/storage analysis	good report, original, good report, original, good report, copy,	N N N T-CSIRC, Vol-	session regarding nuclear safety.  Determines safe limits for four different steps in a manufacturing process.
394 395	NAA-SR-1076  NAA-SR-Memo-4099	A Measurement of the Neutron Temperature Effect Using Europium Oxide Foils  Correction Factors for Measurements with Cadmium Covered Foils  Criticality Criteria for Fabrication of 2% Enriched Uranium Hollow Slugs	A. T. Biehl, E. R. Cohen, D. Woods  D. H. Martin  T. S. Moy	09/25/1951 10/15/1954 07/08/1959	North American Aviation, Inc., Downey CA North American Aviation, Inc., Downey CA Atomics International, North American Aviation, Inc., location not specified  Atomics International, North American Aviation, Inc., location and specified  Atomics International, North American Aviation, Inc., location not specified	Secret, declassified 03/24/1958  [None]  Unclassified, intended for internal use only, may not be published without the approval of AEC.  Unclassified, intended for internal use only, may not be published without the	nuclear measurement/data nuclear measurement/data facility/process/storage analysis	good report, original, good report, original, good report, copy,	N N N T-CSIRC, Vol-	session regarding nuclear safety.  Determines safe limits for four different steps in a manufacturing process.
394 395 396 397	NAA-SR-Memo-4099  NAA-SR-Memo-8195	A Measurement of the Neutron Temperature Effect Using Europium Oxide Foils  Correction Factors for Measurements with Cadmium Covered Foils  Criticality Criteria for Fabrication of 2% Enriched Uranium Hollow Slugs  Criticality Study, 15 w/o U-Zr Alloy  Summary Review of the Kinetics Experiments on	A. T. Biehl, E. R. Cohen, D. Woods  D. H. Martin  T. S. Moy	09/25/1951 10/15/1954 07/08/1959	North American Aviation, Inc., Downey CA North American Aviation, Inc., Downey CA Atomics International, North American Aviation, Inc., location not specified  Atomics International, North American Aviation, Inc., location and specified  Atomics International, North American Aviation, Inc., location and specified	Secret, declassified 03/24/1958  [None]  Unclassified, intended for internal use only, may not be published without the approval of AEC.  Unclassified, intended for internal use only, may not be published without the approval of AEC.	nuclear measurement/data nuclear measurement/data facility/process/storage analysis	good report, original, good report, original, good report, copy, fair report, original,	N  N  T-CSIRC, Vol- 1A	session regarding nuclear safety.  Determines safe limits for four different steps in a manufacturing process.
394 395	NAA-SR-1076  NAA-SR-Memo-4099	A Measurement of the Neutron Temperature Effect Using Europium Oxide Foils  Correction Factors for Measurements with Cadmium Covered Foils  Criticality Criteria for Fabrication of 2% Enriched Uranium Hollow Slugs  Criticality Study, 15 w/o U-Zr Alloy  Summary Review of the Kinetics Experiments on	A. T. Biehl, E. R. Cohen, D. Woods  D. H. Martin  T. S. Moy	09/25/1951 10/15/1954 07/08/1959	North American Aviation, Inc., Downey CA North American Aviation, Inc., Downey CA Atomics International, North American Aviation, Inc., location not specified Atomics International, North American Aviation, Inc., location and the specified Atomics International, North American Aviation, Inc., location and specified Atomics International, North American Aviation, Inc., Canoga	Secret, declassified 03/24/1958  [None]  Unclassified, intended for internal use only, may not be published without the approval of AEC.  Unclassified, intended for internal use only, may not be published without the	nuclear measurement/data nuclear measurement/data facility/process/storage analysis	good report, original, good report, original, good report, copy, fair	N N N T-CSIRC, Vol-	session regarding nuclear safety.  Determines safe limits for four different steps in a manufacturing process.

Γ					Atomics International				1	
ŀ		Material Bucklings of Critical and Subcritical			North American	'				
		Uranium Carbide Fueled Graphite Assemblies			Aviation, Inc., Canoga			report, original,		
400	NAA-SR-9771	1	O. R. Hillig, D. W. Latham	12/01/1964	Park CA	Unclassified	experimental criticality data	good	Y	
400	NAA-3N-37/1	rait1 Experiment	O. K. Hillig, D. W. Latrialli	12/01/1904	Atomics International		experimental criticality data	good	<del> '</del>	
		Material Bucklings of Critical and Subcritical			North American	1				
		Uranium Carbide Fueled Graphite Assemblies			Aviation, Inc., Canoga			report, original,		
401	NAA-SR-9772	Part 2 Theoretical Interpretation	E. R. Specht	08/01/1964	Park CA	Unclassified	experimental criticality data	good	Y	
401	NAA-311-3772	rait 2 Theoretical Interpretation	L. N. Specifi	08/01/1304	Lewis Flight	Officiassified	experimental criticality data	good	·	
					Propulsion					
					Laboratory, National					
					Advisory Committee					
		NACA Zero Power Reactor Facility Hazards	B. Lubarsky, D. J.		for Aeronautics,			report, original,		
402	NACA RM SE57F28	Evaluation	Connolley	06/24/1957	Cleveland OH	[None]	experiment safety analysis	good	N	
	17.107.11111.02.07.12.0	Etaladion	Communicy	00/21/2337	Gievelana Gii	[Hone]	experiment surety unarysis	8000	- "	
1					Lewis Research					
1					Center, National					
					Aeronautics and					
ŀ		Consistent P1 Analysis of Aqueous Uranium-235			Space Administration,		computational method/data	report, original,		
403	NASA TN D-1102	Critical Assemblies	D. Fieno	11/1961	Cleveland OH	[None]	(1)	good	N	
				,			<u> </u>			
1					Lewis Research					
					Center, National					
		Criticality Effects of Centrally Located Tubes and	D. Fieno, E. Gunn, C.		Aeronautics and					
		Rods of Aluminum, Iron, and Tungsten in a	Barber, T. A. Fox, D. L.		Space Administration			report, original,		
404	NASA TN D-1322	Homogeneous Reactor	Alger, R. A. Mueller	08/1962	Cleveland OH	[None]	experimental criticality data	good	Y	
			0.,							
					Lewis Research					
					Center, National					
		Critical Mass Studies with NASA Zero Power			Aeronautics and					
		Reactor II I - Clean Homogeneous	T. A. Fox, R. A. Mueller, C.		Space Administration,			report, original,		
405	NASA TN D-3097	Configurations	H. Ford, D. L. Alger	11/1965	Cleveland OH	[None]	experimental criticality data	good	Y	
					Lewis Research					
					Center, National					
1		Critical Mass Studies with NASA Zero Power			Aeronautics and					
ŀ		Reactor II II - Heterogeneous Arrays of	T. A. Fox, R. A. Mueller, C.		Space Administration,			report, original,		
406	NASA TN D-3555	Cylindrical Voids	H. Ford	08/1966	Cleveland OH	[None]	experimental criticality data	good	Y	
					Lewis Research					
					Center, National					
		Criticality Study of NASA Solution Reactors with			Aeronautics and					
		25.4-Centimeter-Diameter Cylindrical Stainless-	T. A. Fox, R. A. Mueller, D.		Space Administration,	,		report, original,		
407	NASA TM X-2381	Steel Tanks	Fieno	09/1971	Cleveland OH	[None]	experimental criticality data	good	Y	
					Fairchild Engine and					
					Airplane Corp.,					
					Nuclear Energy for					
			F. T. Bly, E. V. Haake, A. O.		the Propulsion of	Secret, declassification				
			Mooneyham, F. W.		Aircraft Project, Oak	indicated but no date		report, original,		NEPA Project: Nuclear Energy for the
408	NEPA 1293-SER-9	Critical Experimentation Status Report	Pressey, G. Thornton	02/16/1950	Ridge TN	provided	experiment plan/design	good	N	Propulsion of Airplanes,
					Fairchild Engine and					
					Airplane Corp.,					
1					Nuclear Energy for					
1					the Propulsion of					13 Pages describing the construction and
						Secret, declassification	1	report, original,		operating characteristics of 2 fission
409	NEPA-1743	Fission Chambers	E. V.Haake (Compiled By:)	04/02/1951	Ridge TN	TID-1268 3-6-62	experiment plan/design	good	J-5700	chambers.
1										
					Fairchild Engine and					Folder also contains a memo (NEPA IC-51
1					Airplane Corp.,					2-7) with a correction to report NEPA-
1					Nuclear Energy for				Y, J-5700	1710 (NEPA-1710 not in Callihan's
			F. T. Bly, J. F. Coneybear,		the Propulsion of					collection). Also, folder contains memos
			E. V. Haake, J. A. Hunter,		Aircraft Project, Oak		experimental criticality data,	1	1	re declassification of NEPA reports and
410	NEPA-1769	NEPA Critical Experiment Facility	H. R. Kroeger	04/15/1951	Ridge TN	02/03/1958	experiment plan/design	good	and C	requests by Callihan for drawings.

					Terrore : T		<del></del>	,		
					Fairchild Engine and					
					Airplane Corp.,					
					Nuclear Energy for					Contains many photographs of the split
					the Propulsion of	Secret, declassification				table hardware (main assembly,
		Electrical and Mechanical Systems (Appendix A	G. Thornton, H. R.		Aircraft Project, Oak	indicated but no date		report, original,		individual components, electrical and
411	NEPA-1827	to NEPA 1769)	Kroeger, D. I. Weinberg	04/15/1951	Ridge TN	provided	experiment plan/design	good	Υ	other subsystems).
						· · · · · · · · · · · · · · · · · · ·				68 Pages of drawings, photos and
					Fairchild Engine and					circuitry for Fission Chambers (NEPA-
					Airplane Corp.,					1743), BF3 Proportional & Ionization
					Nuclear Energy for					(NEPA-1742), Sensory and Photomultron
					the Propulsion of					Chambers(NEPA-1744), various
		The NEDA Critical Experiment Facility (Appendix				Courst dealessification	.	ronart original		
l		The NEPA Critical Experiment Facility (Appendix				Secret, declassification		report, original,		Amplifyiers, Germanium Dioodes and
412	NEPA-1828	B to NEPA 1769) INSTRUMENTS	E. V.Haake (Compiled By:)	04/15/1951	Ridge TN	AEC 7-9-57	experiment plan/design	good	J-5700	Control Panels.
					Fairchild Engine and					
					Airplane Corp.,					
					Nuclear Energy for					Contains photographs of fuel and
					the Propulsion of	Secret, declassification				moderator items, fabrication drawings,
		Materials for Zero Power Reactor Experiments			Aircraft Project, Oak	indicated but no date		report, original,		purity information. Two copies of the
413	NEPA-1829	(Appendix C to NEPA-1769)	F. T. Bly	04/16/1951	Ridge TN	provided	experiment plan/design	good	Υ	report are in the folder.
		Criticality Safety Evaluation of Slightly Enriched						report, copy		
		Uranyl Nitrate Crystals in Vermiculite Shipping	D. L. Dunaway, G. E.					from microcard,		
414	NLCO-1086	Containers	Whitesides	11/03/1971	Fernald	[None]	transport safety analysis	fair	N	
			A. Moat, M. H. McTaggart,	,,	Aldermaston A.E.R.E.	············		report, original,		
415	NR/P-1/59	Discrepancies in v Values	D. S. Mather	04/1959	(U.K.A.E.A.)	Unclassified	nuclear measurements/data	good	N	
413	1411/1-1/33	Discrepancies in V values	D. J. Widther	04/1333	(O.K.A.L.A.)	Officiassified	nuclear measurements/uata	good		
								ronort conu		Translated from German, from the
								report, copy		· ·
		Accidents and Breakdowns at Nuclear			Harwell A.E.R.E.			from microcard,		Karlsruhe Nuclear Research Center
416	NP-tr-1153	Installations	U. Schulze	03/1964	(U.K.A.E.A.)	[None]	criticality accident	poor	N	report KFK 68 (11/1961)
		Safety Analysis of Enriched Uranium Processing	H. T. Williams, J. W.		Convair, General					
		A Study of the Possible Consequences of Nuclear			Dynamics					
}		Accidents in Licensed Plants Processing	H. Wissler, R. E. Fields, M.		Corporation, Forth			report, original,		NYO or NYOO: New York Operations
417	NYO 2980	Unirradiated Enriched Uranium	C. Lawrence	03/18/1960	Worth TX	[None]	criticality accident	good	Υ	Office of the AEC
					American Institutes					
1			R. Fitzpatrick, D. W.		for Research,			report, original,		
418	NYO-3288-10	The Performance of Nuclear Reactor Operators	Dysinger, V. L. Hanson	09/1968	Pittsburg PA	[None]		good	N	
	T	Danger Coefficient for Impurities in Several	1		† <u>-</u>	Secret, declassified		report, original,		
419	NYOO-90	Substances	E. Meservey	01/01/1949	Not specified	02/10/1958	nuclear measurements/data	good	N	
		Further Critical Experiments on a Small Reactor	,			. , .,				
		of Enriched U-235 with Al-H <sub>2</sub> O Moderator and				Secret, declassified		report, original,		
420	00111 70	=		00/45/4040					.,	
420	ORNL-79	Beryllium Reflector	A. B. Martin, M. M. Mann	09/16/1948	ORNL	11/18/1960	experimental criticality data	good	Y	
						Secret, declassification	1 .			
		A Guide with Abstracts to Critical Masses,	A. V. Masket, H. R.			indicated but no date	computational method/data	report, original,		
421	ORNL-112	Laplacians, and Multiplication Factors Part 1	Kroeger	01/17/1949	ORNL	provided	(1)	good	N	
						Secret, declassified		report, original,		
422	ORNL-167	Critical Experiments for the High Flux Reactor	A. B. Martin, M. M. Mann		ORNL	10/19/1960	experimental criticality data	good	Y	
		The Unit Shield Experiments at the Bulk	J. L. Meem, H. E.			Secret, declassified		report, original,		
423	ORNL-1147	Shielding Facility	Hungerford	04/30/1952	ORNL	02/17/1968		good	N	
						Secret, declassified	computational method/data	report, original,		
424	ORNL-1320	The Effect of Gaps on Pile Reactivity	S. Tamor, W. K. Ergen	07/14/1952	ORNL	03/09/1964	(1)	good	N	
			,	, ,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,		Confidential,	<u> </u>			
1						declassified		report, original,		
425	ORNL-1381	A Review of a Polonium Contamination Problem	D. Callihan, D. Ross	08/12/1952	ORNL	07/18/1958		good	N	
	OWAF-1301	The General Methods of Reactor Analysis Used	D. Camman, D. NOSS	00/12/1932	ONNL		computational method/data		IN	
426	00111 4400	1	0.0.450	00/00/46==	0000	Secret, declassified	computational method/data	report, original,		
426	ORNL-1493	by the ANP Physics Group	C. B. Mills	09/22/1953	ORNL	04/22/1963	(1)	good	N	
1		Preliminary Critical Assembly for the Aircraft		1		Secret, declassified		report, original,		
427	ORNL-1634	Reactor Experiment	D. Callihan, D. Scott	10/28/1953	ORNL	07/29/1957	experimental criticality data	good	Y	
			G. S. Hurst, J. A. Harter, P.							
1		Neutron Flux and Tissue Dose Studies with	N. Hensley, W. A. Mills, R.	1		Secret, declassified		report, original,		
428	ORNL-1671	Fission Threshold Detectors	H. Ritchie	03/30/1954	ORNL	02/19/1962	dosimetry	good	N	
					_					

								1		Measurements of multiplication for large
						Secret, downgraded to				cylinders of unmoderated UF <sub>6</sub> at 2%
						· -				,
						Confidential				enrichment. Similar to K-740
		A Task of November 2 Authoritisation by Climbel.				07/09/1956,				experiments except HF was added to
420	ODAU 4600	A Test of Neutron Multiplication by Slightly	A D Callibra	02/46/4054	ODNI	declassified on		report, original,		give H:U-235 = 3.7. The material was in
429	ORNL-1698	Enriched Uranium Part II	A. D. Callihan	03/16/1954	ORNL	10/19/1959	experimental criticality data	good	Υ	liquid state.
		Physics Division Semiannual Progress Report for				Secret, declassified		report, original,		Contains summaries of critical
430	ORNL-1715	Period Ending March 10, 1954	Staff	07/14/1954	ORNL	02/28/1961	multiple topics, TOC scanned	good	Υ	experiments in-progress.
		Preliminary Critical Assemblies of the Reflector				Secret, declassified		report, original,		
431	ORNL-1770	Moderator Reactor	R. M. Spencer	11/22/1954	ORNL	02/08/1962	experimental criticality data	good	Υ	
						Secret, downgraded to				
						Confidential				
						07/11/1956,				
						declassified on		report, original,		
432	ORNL-1726	Critical Mass Studies, Part VII	D. F. Cronin, D. Callihan	06/17/1954	ORNL	08/02/1957	experimental criticality data	good	Y	
		Physics Division Semiannual Progress Report for				Secret, declassified		report, original,		Contains summaries of critical
433	ORNL-1820	Period Ending September 10, 1954	Staff	01/06/1955	ORNL	08/04/1961	multiple topics, TOC scanned	good	Y	experiments in-progress.
			F. C. Maienschein, G. M.							
		Attenuation by Water of Radiations From a	Estabrook, J. D. Flynn, E. B.					report, original,		
434	ORNL-1891	Swimming Pool Type Reactor	Johnson, K. M. Henry	09/07/1955	ORNL	Unclassified	experimental shielding data	good	N	
		Physics Division Semiannual Progress Report for				Secret, declassified		report, original,		Contains summaries of critical
435	ORNL-1926	Period Ending March 10, 1955	Staff	08/23/1955	ORNL	08/02/1957	multiple topics, TOC scanned	good	Y	experiments in-progress.
						Secret, classification				
		Aircraft Nuclear Propulsion Project Quarterly				marks are				
		Progress Report for Period Ending September				obliterated/crossed-		report, original,		
436	ORNL-1947	10, 1955	Staff	10/26/1955	ORNL	out.	multiple topics, TOC scanned	good	Υ	
										Folder contains a memo re classification
										of the report, a memo from Shigekazu
		A Comparison of One-Dimensional Critical Mass								Yoshijima of the Tokyo-Shibaru Electric
		Computations with Experiments for Completely					computational method/data	report, original,		Co., and "spreadsheet" copies for early
437	ORNL-2007	Reflected Reactors	F. G. Prohammer	03/01/1956	ORNL	Unclassified	(1)	good	N	multigroup calculations.
						Confidential,				Folder contains memo about a number
		A Bibliography of ORNL-BSF Reports Pertinent to	F. C. Maienschein, E. B.			declassified	handbook/bibliography,	report, original,		of documents listed in the report being
438	ORNL-2036	Swimming Pool Type Reactor Design	Johnson	04/12/1956	ORNL	10/08/1957	reactor safety	good	N	downgraded for classification.
										Contains no critical experiment
		Physics Division Semiannual Progress Report for						report, original,		summaries; contains pulsed-neutron
439	ORNL-2076	Period Ending March 10, 1956	Staff	06/14/1956	ORNL	Unclassified	multiple topics, TOC scanned	good	Υ	technique and measurement summaries.
		Applied Nuclear Physics Division Annual Report						report, original,		
440	ORNL-2081	for Period Ending September 10, 1956	Staff	11/05/1956	ORNL	Unclassified	multiple topics, TOC scanned	good	Υ	
		Critical Experiments and Nuclear Safety at Oak						report, original,		
441	ORNL-2087	Ridge National Laboratory	A. D. Callihan	08/02/1956	ORNL	Unclassified	experiment plan/design	good	Υ	
		·	D. V. P. Williams, D. W.							
			Magnuson, M. L. Batch,			Confidential,				
			W. R. Johnson, J. K. Leslie,			declassified		report, original,		
442	ORNL-2128	Army Package Power Reactor Critical Experiment		08/08/1956	ORNL	07/29/1957	experimental criticality data	good	Υ	
		1	A. V. H. Masket, R. L.				computational method/data	report, original,		
443	ORNL-2170	Tables of Solid Angles and Activations	Macklin, H. W. Schmitt	11/1956	ORNL	Unclassified	(1)	good	N	
				· · · · · · · · · · · · · · · · · · ·				report, original,		Folder also contains high-quality glossy
444	ORNL-2201	Two Beryllium-Moderated Critical Assemblies	E. L. Zimmerman	10/06/1958	ORNL	[None]	experimental criticality data	good	Υ	photoprints of report figures.
		,				-				Contains no critical experiment
		Physics Division Semiannual Progress Report for						report, original,		summaries; mostly physics and nuclear
445	ORNL-2204	Period Ending September 10, 1956	Staff	01/29/1957	ORNL	Unclassified	multiple topics, TOC scanned	good	Υ	data.
	·	Aqueous Homogeneous Research Reactor -	P. R. Kasten, M. I. Lundin,	, .,				report, original,		
446	ORNL-2256	Feasibility Study	C. L. Segaser	04/10/1957	ORNL	Unclassified	experiment plan/design	good	N	
				1, 25, 255,			- Francisco President	5		Contains no critical experiment
		Physics Division Semiannual Progress Report for						report, original,		summaries; mostly physics and nuclear
447	ORNL-2302	Period Ending March 10, 1957	Staff	06/03/1957	ORNL	Unclassified	multiple topics, TOC scanned	good	Υ	data.
7-77	J*L 2502	. Clos Enaing Waren 10, 1337	J	50/05/1337	OINIEL	Silciussineu	equipment/process design,	BOOG	·····	
							facility/process/storage	report, original,		
448	ORNL-2332	A Criticality Study of the Thorex Pilot Plant	O. O. Yarbro	08/28/1957	ORNL	Unclassified	analysis	good	Υ	
440	OMNL-2332	Critical Mass Studies, Part IX Aqueous U <sup>235</sup>		50/20/133/	OWNE	Onciassineu	ununyaia	-		
440	ORNL-2367	1	J. F. Fox, L. W. Gilley, D. Callihan	02/05/1059	ODNII	Unclassified	experimental criticality data	report, original,	v	
449	UNINL-230/	Solutions	Canilldii	02/05/1958	ORNL	Unclassified	experimental criticality data	good	Y	

		Applied Nuclear Physics Division Annual Report						report, original,		Contains summaries of critical
450	ORNL-2389	for Period Ending September 1, 1957	Staff	10/18/1957	ORNL	Unclassified	multiple topics, TOC scanned	good	Υ	experiments in-progress.
		Standard Operating Procedure for the Pool						report, original,		
451	ORNL-2449	Critical Assembly	E. B. Johnson	08/12/1960	ORNL	[None]	experiment plan/design	good	N	
										Folder includes four internal-ORNL
										memos w details on exposures,
										contamination spread, radiation
										readings, and lessons-learned. Also
		Radiation Excursions at the ORNL Critical								present are a variety of handwritten
		Experiments Laboratory   May 26, 1954   II						report, original,		notes, marked-up building plans, and
452	ORNL-2452	February 1, 1956	J. T. Thomas, A. D. Callihan	05/05/1958	ORNL	[None]	criticality accident	good	Υ	tables of measured data.
						[]		8		
										Folder includes a significant number of
		Boundary Values for the Inner Radius of a					computational method/data	report, original,		handwritten notes, mostly mathematica
453	ORNL-2484	Cylindrical Annular Cylinder	E. L. Zimmerman	06/05/1958	ORNL	[None]	(1)	good	N	equations and computational results.
755	OTTIVE 2404	Comparison of Pool-Type Reactor Critical	E. E. Zimmerman	00/03/1330	OTIVE	[None]	(1)	Bood		equations and computational results.
		Experiments with Two-Group and Thirty-Group					computational method/data	report, original,		
454	ORNL-2499	Calculations	E. G. Silver	05/21/1958	ORNL	[None]	(1)	good	N	
434	OKINL-2455	Calculations	E. G. Silvei	03/21/1936	ORIVE	[None]	(1)	good	IN	Full-scale mockup of the core and
										reflector of Pratt and Whitney Aircraft
										Reactor No. 1. Fuel was a liquid mixture
										of fluoride salts of Na, Zr and enriched L
			D. Scott, G. W. Alwang, E.							
		A Zero Power Reflector-Moderated Reactor	F. Demski, W. J. Fader, E.			Secret, declassified		report, original,		at 1200 to 1350 °F; reflector was Be
455	ORNL-2536	Experiment at Elevated Temperature	V. Sandin, R. E. Malenfant	08/01/1958	ORNL	07/19/1963	experimental criticality data	good	Υ	metal.
		The Effect of Epithermal Fission on Aqueous					computational method/data	report, original,		
456	ORNL-2553	Homogeneous Reactors	S. Terasawa	07/23/1958	ORNL	[None]	(2)	good	N	
		Review of Experimental and Theoretical Studies								
		Concerning the Age of Fission Neutrons on					computational method/data	report, original,		
457	ORNL-2575	Heavy Water	M. Tobias	10/01/1958	ORNL	[None]	(1)	good	N	
		Neutron Physics Division Annual Report for						report, original,		Contains summaries of critical
458	ORNL-2609	Period Ending September 1, 1958	Staff	10/18/1957	ORNL	[None]	multiple topics, TOC scanned	good	Υ	experiments in-progress.
		Theory of Resonance Absorption of Neutrons in					computational method/data	report, original,		
459	ORNL-2705	a Lump	K. Hasegawa	06/02/1959	ORNL	[None]	(1)	good	N	
		Dump Tank Criticality and Poisons for Slurry					computational method/data	report, original,		For H <sub>2</sub> O or D <sub>2</sub> O-moderated reactors
460	ODNI 2700		D. F. Drings	06/26/1050	ORNL	[None]		1	N	fueled with U <sup>235</sup> , designed to breed U <sup>235</sup>
460	ORNL-2708	Reactors	B. E. Prince	06/26/1959	URINL	[None]	(1), equipment/process design	good	N	lueled with 0 , designed to breed 0
										Most papers deal with computational
		B 11 51 N 1 T 1 1 1								Most papers deal with computational
		Proceedings of the Neutron Thermalization								methods or experimental measurement
		Conference, April 28-30, 1958 Gatlinburg,						report, original,		of neutron slowing-down in reactors or
461	ORNL-2739	Tennessee	Various Authors	09/02/1959	ORNL	[None]	multiple topics, TOC scanned	good	N	particular materials.
		Description of the Tower Shielding Reactor II and					experimental criticality data,	report, original,		
462	ORNL-2747	Proposed Preliminary Experiments	L. B. Holland, C. E. Clifford	07/06/1959	ORNL	[None]	experiment plan/design	good	Υ	
		Radiation Accidents: Dosimetric Aspects of						report, original,		
463	ORNL-2748	Neutron and Gamma-Ray Exposures	G. S. Hurst, R. H. Ritchie	11/02/1959	ORNL	Unclassified	criticality accident, dosimetry	good	Y	
		The Fast-Neutron Multiplication Effect of					computational method/data	report, original,		
464	ORNL-2779	Beryllium in Reactors	W. Häfele	09/08/1959	ORNL	[None]	(1)	good	N	
		Neutron Physics Division Annual Report for						report, original,		Contains summaries of critical
465	ORNL-2842	Period Ending September 1, 1959	Staff	11/09/1959	ORNL	[None]	multiple topics, TOC scanned	good	Y	experiments in-progress.
					-			report, original,		
466	ORNL-3006	HFIR Preliminary Physics Report	R. D. Cheverton	10/04/1960	ORNL	[None]	reactor safety	good	Υ	
		Neutron Physics Division Annual Report for						report, original,		Contains summaries of critical
467	ORNL-3016	Period Ending September 1, 1960	Staff	11/30/1960	ORNL	[None]	multiple topics, TOC scanned	good	Υ	experiments in-progress.
		Effect of Heat Flux on the Corrosion of								
		Aluminum by Water. Part II. Influence of Water	J. C. Griess, H. C. Savage,							
		Temperature, Velocity, and pH on Corrosion-	T. H. Mauney, J. L. English,					report, original,		
468	ORNL-3056	Product Formation	J. G. Rainwater	02/10/1961	ORNL	[None]	operational/test/material data	1	N	
		The Corrosion of Aluminum Alloys in High-	J. L. English, L. Rice, J. C.	, .,			, , ,	report, original,	-	
469	ORNL-3063	Velocity Water at 170 to 290 °C	Griess	06/01/1961	ORNL	[None]	operational/test/material data		N	
403	CHINE-2002	Neutron Physics Division Annual Report for	011033	50/01/1501	ONNE	[MOHE]	operational/test/material udta	report, original,	ıN	Contains summaries of critical
470	ORNL-3193	Period Ending September 1, 1961	Staff	10/31/1961	ORNL	[None]	multiple topics, TOC scanned	good	Υ	experiments in-progress.
4/0	OUMF-2132	renou chaing september 1, 1961	Stall	10/21/1701	UKNL	[None]	multiple topics, TOC scanned	goou	f	experiments in-progress.

							T			
		Effect of Heat Flux on the Corrosion of	J. C. Griess, H. C. Savage, J.							
		Aluminum by Water. Part III. Final Report on	G. Rainwater, T. H.					report, original,		
471	ORNL-3230	Tests Relative to the High-Flux Isotope Reactor	Mauney, J. L. English	12/05/1961	ORNL	[None]	operational/test/material data	good	N	
		Health Physics Research Reactor Hazards						report, original,		
472	ORNL-3248	Summary	M. I. Lundin	08/24/1962	ORNL	[None]	experiment safety analysis	good	Υ	
						Confidential,				
		Critical Mass Studies: Part XI. Critical				declassified		report, original,		
473	ORNL-3272	Parameters of Uranium-Aluminum Alloy Slugs	J. K. Fox, L. W. Gilley	05/14/1962	ORNL	10/14/1966	experimental criticality data	good	Υ	
		Soluble Neutron Poisons as a Primary Criticality								
		Control in Shielded and Contained					computational method/data	report, original,		
474	ORNL-3309	Radiochemical Facilities	J. P. Nichols	07/12/1962	ORNL	[None]		good	Υ	
4/4	UNINE-3303		J. F. INICIIOIS	07/12/1902	ORIVE	[None]	(1), equipment/process design		······	
	00111 2240	Nuclear Safety Program Semiannual Progress	C) II	00/47/4000	00111	ra. 1		report, original,	.,	
475	ORNL-3319	Report for Period Ending June 30, 1962	Staff	08/17/1962	ORNL	[None]	multiple topics, TOC scanned	good	Y	
		Neutron Physics Division Annual Report for	-					report, original,		Contains summaries of critical
476	ORNL-3360	Period Ending September 1, 1962	Staff	10/08/1962	ORNL	[None]	multiple topics, TOC scanned	good	Υ	experiments in-progress.
			R. W. Horton, D. W.							
		Criticality Analysis: LWBR Assistance Program in	Magnuson, W. T.				computational method/data	report, original,		
477	ORNL-TM-3469	Building 3019	McDuffee	03/1972	ORNL	[None]	(2), equipment/process design	good	Υ	
		Neutron Physics Division Annual Report for						report, original,		Contains summaries of critical
478	ORNL-3499, Vol 1	Period Ending August 1, 1963	Staff	10/21/1963	ORNL	[None]	multiple topics, TOC scanned	good	Υ	experiments in-progress.
		Neutron Physics Division Annual Report for						report, original,		
479	ORNL-3499, Vol 2	Period Ending August 1, 1963	Staff	10/30/1963	ORNL	[None]	multiple topics, TOC scanned	good	N	
· · · · · · · · · · · · · · · · · · ·	,			.,,		+		f · · ·		Folder includes a document titled "HPRR
		Procedures Manual for the Health Physics								Operating Procedures for Operation
		Research Reactor (Including High-Level Gamma						report, original,		BREN," no report number or cover,
400	ODNU 3540		C+-44	40/24/4062	ORNU	[Na1		1	V	· '
480	ORNL-3519	Facility)	Staff	10/24/1963	ORNL	[None]	experiment plan/design	good	Υ	dated 01/25/1962
		O5R, A General-Purpose Monte Carlo Neutron	D. C. Irving, R. M.				computational method/data	report, original,		
481	ORNL-3622	Transport Code	Freestone Jr., F. B. K. Kam	02/1965	ORNL	[None]	(2)	good	Υ	
l										Contains summaries of critical
ŀ		Nuclear Safety Program Semiannual Progress						report, original,		experiments in-progress (E. B. Johnson's
482	ORNL-3691	Report for Period Ending June 30, 1964	Staff	11/1964	ORNL	[None]	multiple topics, TOC scanned	good	Υ	"1-D Reactor" series).
		Neutron Physics Division Annual Report for						report, original,		Contains summaries of critical
483	ORNL-3714, Vol. I	Period Ending August 1, 1964	Staff	12/1964	ORNL	[None]	multiple topics, TOC scanned	good	Υ	experiments in-progress.
		Neutron Physics Division Annual Report for						report, original,		
484	ORNL-3714, Vol. II	Period Ending August 1, 1964	Staff	12/1964	ORNL	[None]	multiple topics, TOC scanned	good	N	
	, , ,	Speculations on the Interpretation of Neutron		,			computational method/data	report, original,		
485	ORNL-3757	Noise Experiments	R. K. Osborn	01/1965	ORNL	[None]	(2)	good	N	
						[]		1		
										Summary of ORNL activities during 1963,
								roport original		_ ·
400	ODNII 3650	Only Bidge Matingal Laboratory 1063	C+-#	02/1005	ORNU	[Na1		report, original,		including critical experiments, Oak Ridge
486	ORNL-3650	Oak Ridge National Laboratory 1963	Staff	02/1965	ORNL	[None]		good	N	School of Reactor Technology.
		Neutron Physics Division Annual Report for						report, original,		Contains summaries of critical
487	ORNL-3858, Vol. I	Period Ending August 1, 1965	Staff	11/1965	ORNL	[None]	multiple topics, TOC scanned	good	Υ	experiments in-progress.
		Neutron Physics Division Annual Report for						report, original,		
488	ORNL-3858, Vol. II	Period Ending August 1, 1965	Staff	11/1965	ORNL	[None]	multiple topics, TOC scanned	good	N	
		Neutron Physics Division Annual Report for						report, original,		Contains summaries of critical
489	ORNL-3973, Vol. I	Period Ending May 31, 1966	Staff	09/1966	ORNL	[None]	multiple topics, TOC scanned	good	Υ	experiments in-progress.
		Neutron Physics Division Annual Report for						report, original,		
490	ORNL-3973, Vol. II	Period Ending May 31, 1966	Staff	09/1966	ORNL	[None]	multiple topics, TOC scanned	good	N	
		Adjoint and Importance in Monte Carlo	R. R. Coveyou, V. R. Cain,			i	computational method/data	report, original,		
491	ORNL-4093	Application	K. J. Yost	04/1967	ORNL	[None]	(2)	good	N	
<del></del>		Neutron Physics Division Annual Report for		2 ., 250,		,,	1-1	report, original,	•••	Contains summaries of critical
492	ORNL-4134	Period Ending May 31, 1967	Staff	08/1967	ORNL	[None]	multiple topics, TOC scanned	good	Υ	experiments in-progress.
474	OMAL-4134		Jean	00/150/	ONNE	[.vone]	multiple topics, TOC scallied	Бооп	·	experiments in-progress.
		Evaluation of the Two-Detector Cross-	D C K-+ C ** 5 - 5 -					L		
	00111 4255	Correlation Technique for Shutdown Margin	R. C. Kryter, D. N. Fry, D. P.	00/4	05	fa. 3	computational method/data	report, original,		
493	ORNL-4255	Measurements of Power Reactors	Roux	09/1968	ORNL	[None]	(2)	good	N	
		Critical Experiments for the Repetitively Pulsed						report, original,		
494	ORNL-4263	Reactor SORA	G. Kistner, J. T. Mihalczo	06/1968	ORNL	[None]	experimental criticality data	good	Υ	
		Neutron Physics Division Annual Report for						report, original,		Contains summaries of critical
495	ORNL-4280	Period Ending May 31, 1968	Staff	10/1968	ORNL	[None]	multiple topics, TOC scanned	good	Υ	experiments in-progress.
										Contains discussions (and certain details)
			R. D. Cheverton, T. M.					report, original,		about the critical experiments and
496	ORNL-4621	HFIR Core Nuclear Design	Sims	07/1971	ORNL	[None]	reactor safety	good	Υ	decisions based on experiment results.
.50	1		r	,, -	2	1,	1	10.00	•	experiment results.

		Neutron Physics Division Annual Report for						report, original,		Contains summaries of critical
497	ORNL-4705	Period Ending May 31, 1971	Staff	10/1971	ORNL	[None]	multiple topics, TOC scanned	good	N	experiments in-progress.
		Calculated Criticality Data for LMFBR (U+Pu)O2 -					computational method/data	report, original,		
498	ORNL-4711	Water Systems	W. R. Cobb	01/1972	ORNL	[None]	(2)	good	N	
		KENO IV An Improved Monte Carlo Criticality					computational method/data	report, original,		
499	ORNL-4938	Program	L. M. Petrie, N. F. Cross	11/1975	ORNL	[None]	(2)	good	Υ	
		Reactivity Calibrations and Fission-Rate  Distributions in an Unmoderated, Unreflected				not to be given public dissemination		report, original,		Critical experiments to support operation of the ORNL Health Physics
500	ORNL-TM-189	Uranium-Molybdenum Alloy Research Reactor	J. T. Mihalczo	05/10/1962	ORNL	without approval	experimental criticality data	good	Υ	Research Reactor.
						not to be given				
		An Estimate of the Effect of Neutron-Energy				public dissemination		report, original,		
501	ORNL-TM-299	Spectrum on Radiation Damage of Steel	H. C. Claiborne	07/27/1962	ORNL	without approval	operational/test/material data	good	N	Folder includes a marked-up report draft
										and related paperwork, also a copy of
						prepared primarily				BNL 6415, "The Chemical and Physical
		A Safety Analysis of the Oak Ridge Critical				for internal use at the	experiment safety analysis,	report, original,		Behavior of Released Fission Products,"
502	ORNL-TM-349, Rev. 1	Experiments Facility	Staff	02/1967	ORNL	ORNL	experiment plan/design	good	Υ	A. W. Castleman, Jr.
										Due to the limited availability of fuel assemblies, the maximum multiplication
										achieved was ~ 5. The experiments did
						not to be given				determine the most reactive spacing for
		Neutron Multiplication by Experimental Gas	E. B. Johnson, R. K. Reedy			public dissemination		report, original,		water-immersed lattices of assemblies
503	ORNL-TM-433	Cooled Reactor Fuel Assemblies	Jr.	11/27/1962	ORNL	without approval	experimental criticality data	good	Y	(touching).
						Confidential, declassified				
						09/04/1968. not to be				
						given public				
		A Small Graphite-Reflected UO <sub>2</sub> Critical				dissemination without		report, original,		
504	ORNL-TM-450	Assembly	J. T. Mihalczo	12/28/1962	ORNL	approval	experimental criticality data	good	Y	
		Prompt Neutron Decay in a Two-Component				not to be given public dissemination		report, original,		
505	ORNL-TM-470	Enriched Uranium Metal Critical Assembly	J. T. Mihalczo	01/11/1963	ORNL	without approval	experimental criticality data	good	Υ	
		Critical Mass Studies - Part XIII Borosilicate				not to be given				
	00111 T14 405	Glass Raschig Rings in Aqueous Uranyl Nitrate	J. T. Thomas, J. K. Fox, E. B.	00/05/		public dissemination		report, original,		
506	ORNL-TM-499	Solutions	Johnson	02/06/1963	ORNL	without approval	experimental criticality data	good	Y	Report contains pencil and ink notes.  Describes various administrative
						not to be given				actions/policies and facility upgrades
		Radiation Safety and Control at the Oak Ridge				public dissemination		report, original,		motivated by the SL-1 accident and
507	ORNL-TM-507	National Laboratory: 1960-1962	F. R. Bruce	04/05/1963	ORNL	without approval	criticality accident	good	N	ORNL radiological events.
		Source Strongth and Lore Source Deep				not to be given		manaut saisiasi		
508	ORNL-TM-550	Source-Strength and Long-Counter-Response Calibrations at the BSF 300-kev Accelerator	E. G. Silver	04/08/1963	ORNL	public dissemination without approval		report, original, good	N	
300	GIMAE-TIMI-330	Campitations at the DSI SOUTREY Accelerator	E. G. JIIVEI	04/00/1303	ONINE	Confidential,	<u> </u>	Боод	IN	
						declassified				
						09/04/1968. not to be				
		A Small Graphite-Reflected UO <sub>2</sub> Critical				given public				The copy in the folder is a B&W
509	ORNL-TM-561	Assembly, Part II	J. T. Mihalczo	04/08/1963	ORNL	dissemination without approval	experimental criticality data	report, copy, fair	Υ	photocopy of the copy issued to J. T. Mihalczo.
202	OTTAL LIAI-201	proseniory, rate ii	J. I. WIIIIaiczo	J4/ JJ/ 13U3	ONNE	approval	Caperiniental criticality udid	rull	'	iviiiidiczo.

The Reactor Transients Accompanying Pulsed Source Operation and Step Changes of Reactivity C. A. Presitit  Present Computation and Step Changes of Reactivity of The Auditing of Reactor Safety at the Oak Ridge S11 ORNL-TM-512 National Laboratory  Present Computation of Reactivity of The Auditing of Reactor Safety at the Oak Ridge S12 ORNL-TM-515 National Laboratory  Present Computation of Reactivity of The Computation of Reactivity of The Computation of Reactivity of The Computation of Reactivity of National Laboratory  Present Computation of Reactivity of National Laboratory  Present Computation of Reactivity of National Reactor Safety at the Oak Ridge National Laboratory Reactor Safety Systems National Laborat		1					Not to be given		т т		
10   10   10   11   11   12   13   13   13   14   14   14   14   15   14   15   14   15   15			The Peactor Transients Assembanying Bulsed				Not to be given	computational method/data	roport original		
Part	F10	ODNI TM E70		C A Drockitt	04/19/1063	ODNI	F .		1	N	
Mark 174-52   National Country   February 18 per	510	UKNL-TWI-570	Source Operation and Step Changes of Reactivity	C. A. Preskitt	04/18/1963	URNL		(1)	good	IN	
Marcon Laberatory   Part   P			The Audition of Beauty Cofety at the Collaboration								
Open		ODAN TA 642		- " .	07/00/4060		f .		1		
Operating Procedures for the Hold holder   No. Herry 1, D. Pariglico,   College	511	ORNL-1M-612	National Laboratory	F. Kertesz	07/08/1963	ORNL		safety review	good	N	
1											
13   ORR.TM-55			1		/ /		IF.				
A	512	ORNL-TM-616	Reactor Fifth Edition	F. C. Maienschein	07/15/1963	ORNL		experiment plan/design	ļ		
11   12   13   13   13   14   14   15   15   15   15   15   15											
A Small Reyllum-lifected UC, Assembly J. T. Milhalaro 0772/1916 ORN. 1. approval performed artically data for the Code of the											
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A Custe to the Design of Shigging Casis for the Shigger of Shigging Casis for the Shigging Cas							dissemination without		report, copy,		photocopy of the copy issued to
A Coulde to the Design of Subgrang Casks for the Pursupor of Subgrang Casks for the Pursupor of Subgrang Casks for the Pursupor of Subgrang Casks for the ext. 20	513	ORNL-TM-655	A Small Beryllium-Reflected UO <sub>2</sub> Assembly	J. T. Mihalczo	07/23/1963	ORNL	approval	experimental criticality data	fair	Υ	Laboratory Records.
A Guide to the Design of Shopping Cashs or the Transported Medical B. B. Shappert Out; 1955 ORNI. Whitton approval epigenetrative analysis. Papert, original, a plant to be a given to to be a given to the Health Physics Research Reactor W. Williams (1975) ORNI. Whitton approval public dissemination white feet of Various Reflectors on the Reactivity of the Health Physics Research Reactor W. College Original Basic State College Original Basic S											Contains discussions regarding early
Section   Sect							not to be given				1960s regulations for transport and
Section   Sect			A Guide to the Design of Shipping Casks for the				public dissemination	transport safety analysis,	report, original,		shipping containers of the era, ~ 20
sfets of various Refection on the Reactivity of the Health Physics Research Reactor   Critical Three Dimensional Array of Neutron.  Critical Three Dimensional Neutr	514	ORNL-TM-681	1	L. B. Shappert	04/1965	ORNL	IF.	1		N	
Control   Cont					0 1, 2000				8		F-6
ONN_THY-710 the Health Physicia Research Reactor Services (Parks 1) (1971-1976) and provided in the Health Physicia Research Reactor Services (Parks 1) (1971-1976) and provided in the Health Physicia Research Reactor Services (Parks 1) (1971-1976) and provided in the Health Physicia Research Reactor Services (Parks 1) (1971-1976) and Parks 1971-1976 and Parks 1971			Effects of Various Reflectors on the Reactivity of						report original		
Critical Three-Dimensional Arrays of Neutron Interacting Units  Density Company of Neutron Interacting Units  Dens	515	ORNI -TM-710	1	I W Gilley	02/1964	ORNI	f .	evnerimental criticality data	1 1	N	
Critical Tree-Dimensional Arrays of Neutron  Descripting Experience with Coincident Vs. Non-coincident Reacted Safety Systems  Associates and Reflection on the Reactivity of  Associates and Reflection on the Reactivity of the Register of the Computer of the Reflection of the Register of the Register of the Reflection of the Register of the Register of the Reflection Spectral and Reflection of the Register of the Re	313	ORIVE-TIVI-710	the freath rifysics research reactor	L. W. Officy	02/1304	ONNE		experimental criticality data	good		
516   ORN-TW-719   Information Certain Reactivity   Ornal			Critical Thron Dimonsional Arrays of Noutron						roport original		
Operating Experience with Coincident Vs. 17 Operating Experience with Coincident Vs. 18 Operating Experience w	F16	ODNI TNA 740		I T Thomas	10/01/1063	ODNI	ľ	avantimontal criticality data	1	V	
Operating Experience with Coincident Vs. No. Noncincident Reactivity of Aborters and Reflection of the Reactivity of Aborters and Reflection on the Reactivity of Aborters and Reflection of Reactivities and Reflection of Reactivities and Reflection of Reactivities and Reflection of Reacti	210	ORINL-11VI-719	interacting Onits	J. I. IIIOIIIdS	10/01/1903	URNL		experimental criticality data	good		
ONN_TAY-788   Noncoincident Reactor Safety Systems   F. Epier   12/12/1963   ONN_ without _approval _ not to be _given _ public dissernation _ (1), equipment/process design _ good   N											
Absorber and Refectors on the Reactivity of Absorber and Refectors on the Reactivity of Solutions of <sup>230</sup> pu <sup>240</sup> pu in Slugs			1				IF.	_	1		
Absorbers and Reflectors on the Reactivity of Solutions of 22 Pu-22 Pu in Stage	517	ORNL-TM-738	Noncoincident Reactor Safety Systems	E. P. Epler	12/12/1963	ORNL	without approval	reactor safety	good	N	
Solutions of <sup>228</sup> Pu- <sup>248</sup> Pu in Slugs    No.   N			1				not to be given				
Second Description   Formation Centers at the Oak Ridge National Laboratory (A Study in Diversity)   F. Kertesz   11/13/1964   ORNL			Absorbers and Reflectors on the Reactivity of				public dissemination	computational method/data	report, original,		
Information Centers at the QaR Ridge National Laboratory (A Study in Diversity)  Laboratory (A Study in Diversity)  Results (A Laboratory (A Labor	518	ORNL-TM-821	Solutions of <sup>239</sup> Pu- <sup>240</sup> Pu in Slugs	J. P. Nichols	03/31/1964	ORNL	without approval	(1), equipment/process design	good	N	
Sender Members of Members of Members of Sender Members of Memb							not to be given				Discusses planned creation of several
Soundary Conditions for the Cylindricalized Cell of Reactor Lattice Calculations  All W. Webster 12/30/1964 ORNL  Measurement of the "SIU Neutron Capture to-Fission Ratio, for Incident Neutron Energies  Fission Ratio, for Incident Neutron Energies  G. de Saussure, L. W.  Weston, R. Gwin  O1/26/1965 ORNL  TW. Hanauer, E. R. Mann  O4/1965 ORNL  ORNL-TM-1066 and Subcritical Reactivity  Interpretation of Pulsed Source Experiments  Nephew O4/27/1965 ORNL  OPERATION For State Structure S			Information Centers at the Oak Ridge National				public dissemination		report, original,		technical "centers" at ORNL, including
Boundary Conditions for the Cylindricalized Cell of Reactor Lattice Calculations  Measurement of the "FW Neutron Capture to-Fission Ratio, for Incident Neutron Energies  All Capture Control of Reactor Lattice Calculations  Measurement of the "FW Neutron Energies  S21 ORNL-TM-1041 from 3.25 eV to 1.8 keV  Measurement of Reactor Fluctuation Spectra  And Subscritical Reactivity  Measurement of Plused Source Experiments  Increased Vielia From Fast Burst Reactors. Part.  Effects of Increased Mass on Uranium and Uranium-Molydenum critical Cipidners  Operating Experience of the Nuclear Safety Information Center March 1963 - March 1965  ORNL-TM-1125  Operating Experience of the Nuclear Safety Information of Neutron Flux in Nuclear Reactor by the Uncollided-Flux Estimator  Applied to Month Carlo Collisions  J. R. Buchanan, W. B. OS/07/1965  ORNL Milhalczo  OS/07/1965  ORNL Milhalczo  OS/07/1965  ORNL Milhalczo  ORNL Milhalczo  OS/07/1965  ORNL ORNL Deproval.  OS/07/1965  ORNL ORNL Deproval.  OS/07/1965  ORNL Milhalczo  OS/07/1965  ORNL ORNL Deproval.  OS/07/1965  ORNL ORNL Deproval	519	ORNL-TM-996	Laboratory (A Study in Diversity)	F. Kertesz	11/13/1964	ORNL	without approval		good	N	the Criticality Data Center.
Soundary Conditions for the Cylindricalized Cell   I. W. Webster   12/30/1964   ORNL   without a sproval   Conditions   I. W. Webster   12/30/1964   ORNL   without a sproval   Conditions   I. W. Webster   12/30/1964   ORNL   without a sproval   Conditions   I. W. Webster   12/30/1965   ORNL   without a sproval   Conditions   I. W. Webster   I. W.					, ,				ř		· · · · · · · · · · · · · · · · · · ·
S20 ORNL-TM-1022 of Reactor Lattice Calculations   J. W. Webster   12/30/1964 ORNL   without approval   (1)   good   N    Measurement of the 2 <sup>15t</sup> U Neutron Capture-to-Fission Rato, a, for Incident Neutron Energies   G. de Saussure, L. W.    S21 ORNL-TM-1041 from 3.25 eV to 1.8 keV   Weston R. G. Win   West			Boundary Conditions for the Cylindricalized Cell					computational method/data	report, original,		
Measurement of the 250 Newtron Capture-to-Fision Ratio, $a_i$ , for Inddern Neutron Energies of G. de Saussure, L. W. Weston, R. Gwin 01/26/1965 ORNL without approval nuclear measurement/data good N report, original, report, original, and Subcritical Reactivity hanauer, E. R. Mann 04/1965 ORNL [None] experimental criticality data good N report, original, report,	520	ORNL-TM-1022		J. W. Webster	12/30/1964	ORNL	f .			N	
Fission Ratio, <i>u</i> , for Incident Neutron Energies G. de Saussure, L. W. Weston, R. Gwin (Messurement of Reactor Fluctuation Spectra and Subcritical Reactivity (Messurement Spectra and Spectra and Subcritical Reactivity (Messurement Spectra and Spectra					, ,	-					
S21   ORNL-TM-1041   from 3.25 eV to 1.8 keV   Meston, R. Gwin   Ol/26/1965   ORNL   without approval   nuclear measurement/data   good   N			-	G de Saussure I W					report original		
Measurement of Reactor Fluctuation Spectra And Subcritical Reactivity Annual Reactiv	F21	ODNII TNA 1041	_	,	01/26/1065	ODNI	f .	nuclear massurement /data	1	N	
S22 ORNL-TM-1066 and Subcritical Reactivity Hanauer, E. R. Mann 04/1965 ORNL [None] experimental criticality data good N  C. A. Preskitt, E. A. Nephew 04/27/1965 ORNL [None] experimental criticality data good N  Increased Yields from Fast Burst Reactors. Part I. Effects of Increased Mass on Uranium and Increased Yields from Fast Burst Reactors. Part I. Effects of Increased Mass on Uranium and Uranium and Operating Experience of the Nuclear Safety Uranium-Molybdenum Critical Cylinders  Operating Experience of the Nuclear Safety Information Center March 1963 - March 1965  ORNL-TM-1136 Information OR Neutron Flux in Nuclear Safety (Control of Nuclear Safety)  ORNL-TM-1136 Information Center March 1963 - March 1965  ORNL-TM-1136 ORNL-TM-1157 Applied to Monte Carlo Collisions  J. W. Webster C. B. J. W. Webster C. B. J. W. Webster C. B. J. W. Webster, E. B. Johnson O7/23/1965 ORNL ORNL Center March 1965 ORNL ORNL Center March 1967 Orn Internal use at the Computational method/data report, original, report,	521	ORINL-11VI-1U41			01/26/1905	OKNL	without approval	nuclear measurement/data		IN	
C. A. Preskitt, E. A. Nephew 04/27/1965 ORNL [None] experimental criticality data good information regarding subcritic measurement theory - not clear what near the control of Pulsed Source Experiments increased Vields from Fast Burst Reactors. Part L. Effects of Increased Nass on Uranium and Uranium and Uranium-Molybdenum Critical Cylinders J. T. Mihalczo 04/30/1965 ORNL without approval reactor safety good N. Operating Experience of the Nuclear Safety information Center March 1963 - March 1965 ORNL without approval report, original, good N. Mostly describes activities similar to those of a technical library.  The Determination of Neutron Flux in Nuclear Reactor by the Uncollided-Flux Estimator Applied to Monte Carlo Collisions J. W. Webster 08/23/1965 ORNL ORNL prepared primarily for internal use at the SOURCE, a Neutron Distribution Routine for the SOURCE, a Neutron Distribution Routine for the Source Criticality of a Single Unit of Aqueous Uranyl Flux in Nuclear Source Criticality of a Single Unit of Aqueous Uranyl Flux in Nuclear Source Criticality of a Single Unit of Aqueous Uranyl Flux in Nuclear Source Criticality of a Single Unit of Aqueous Uranyl Flux in Nuclear Source Criticality of a Single Unit of Aqueous Uranyl Flux in Nuclear Source Criticality of a Single Unit of Aqueous Uranyl Flux in Nuclear Source Criticality of a Single Unit of Aqueous Uranyl Flux in Nuclear Source Criticality of a Single Unit of Aqueous Uranyl Flux in Nuclear Source Criticality of a Single Unit of Aqueous Uranyl Flux in Nuclear Source Criticality of a Single Unit of Aqueous Uranyl Flux in Nuclear Source Criticality of a Single Unit of Aqueous Uranyl Flux in Nuclear Source Criticality of a Single Unit of Aqueous Uranyl Flux in Nuclear Source Criticality of a Single Unit of Aqueous Uranyl Flux in Nuclear Source Criticality of a Single Unit of Aqueous Uranyl Flux in Nuclear Source Criticality of a Single Unit of Aqueous Uranyl Flux in Nuclear Source Criticality of a Single Unit of Aqueous Uranyl Flux in Nucle	500	ODAU TA 4000			04/4065		fa. 3		1		
C. A. Preskitt, E. A. Nephew O4/27/1965 ORNL [None] experimental criticality data good N measurement theory - not clear what is unauthorized increased Yields from Fast Burst Reactors. Part I. Effects of Increased Mass on Uranium and Uranium-Molybdenum Critical Cylinders J. T. Mihalczo O4/30/1965 ORNL without approval reactor safety good N measurement theory - not clear what measured data was employed.  Departing Experience of the Nuclear Safety Operating Experience of the Nuclear Safety of Determination of Neutron Flux in Nuclear Reactor by the Uncollided-Flux Estimator Applied to Monte Carlo Collisions J. W. Webster O8/23/1965 ORNL ORNL prepared primarily for internal use at the OS Monte Carlo Code ORNL OS Monte Carlo Code ORNL ORNL prepared primarily for internal use at the Computational method/data (2) good N ORNL prepared primarily for internal use at the Computational method/data (2) good N ORNL prepared primarily for internal use at the Computational method/data (2) good N ORNL prepared primarily for internal use at the Computational method/data (2) good N ORNL prepared primarily for internal use at the Computational method/data (2) good N ORNL prepared primarily for internal use at the Computational method/data (2) good N ORNL prepared primarily for internal use at the Computational method/data (2) good N ORNL prepared primarily for internal use at the Computational method/data (2) good N ORNL prepared primarily for internal use at the Computational method/data (2) good N ORNL prepared primarily for internal use at the Computational method/data (2) good N ORNL prepared primarily for internal use at the Computational method/data (2) good N ORNL prepared primarily for internal use at the Computational method/data (2) good N N ORNL prepared primarily for internal use at the Computational method/data (2) good N N ORNL prepared primarily for internal use at the Computational method/data (2) good N N ORNL prepared primarily for internal use	522	ORNL-1M-1066	and Subcritical Reactivity	Hanauer, E. R. Mann	04/1965	ORNL	[None]	experimental criticality data	good	N	
S23   ORNL-TM-1113   Interpretation of Pulsed Source Experiments   Nephew   O4/27/1965   ORNL   (None)   experimental criticality data   good   N   measured data was employed.											
Increased Yields from Fast Burst Reactors. Part I. Effects of Increased Mass on Uranium and Uranium Analybdenum Critical Cylinders Uranium-Molybdenum Critical Cylinders J. T. Mihalczo 04/30/1965 ORNL without approval reactor safety good N  Operating Experience of the Nuclear Safety Information Center March 1963 - March 1965 Cottrell O5/07/1965 ORNL without approval prepared primarily for internal use at the OSRNL ORNL ORNL prepared primarily for internal use at the OSRNL prepar									1		•
Effects of Increased Mass on Uranium and Uranium-Molybdenum Critical Cylinders  Departing Experience of the Nuclear Safety Operating Public dissemination on to to be given public dissemination on the without approval Operating Experience of the Nuclear Safety Operating Public dissemination on the without approval Operating Experience of the Nuclear Safety Operating Public dissemination on the without approval Operating Experience of the Nuclear Safety Operating Public dissemination on the without approval Operating Public dissemination of the witho	523	ORNL-TM-1113		Nephew	04/27/1965	ORNL			good	N	measured data was employed.
ORNL-TM-1125 Uranium-Molybdenum Critical Cylinders J. T. Mihalczo 04/30/1965 ORNL without approval reactor safety good N  Operating Experience of the Nuclear Safety Operating Experience of the Nuclear Safety Information Center March 1963 - March 1965  ORNL-TM-1136 Information Center March 1963 - March 1965  The Determination of Neutron Flux in Nuclear Reactor by the Uncollided-Flux Estimator Applied to Monte Carlo Collisions  J. W. Webster 08/23/1965 ORNL ORNL prepared primarily for internal use at the SURCE, a Neutron Distribution Routine for the OSM Morrison, D. Irving O7/08/1965 ORNL ORNL prepared primarily for internal use at the OSM Nuclear Normalization of Neutron Flux in Nuclear Reactor by the Uncollided-Flux Estimator Applied to Monte Carlo Colde Morrison, D. Irving O7/08/1965 ORNL ORNL prepared primarily for internal use at the OSM Nuclear Normalization of Neutron Distribution Routine for the Morrison, D. Irving O7/08/1965 ORNL ORNL prepared primarily for internal use at the OSM Nuclear Normalization of Neutron Distribution Routine for the Normalization ORNL prepared primaril			1								
Operating Experience of the Nuclear Safety Information Center March 1963 - March 1965 ORNL-TM-1136 ORNL-TM-11			Effects of Increased Mass on Uranium and				is unauthorized		report, original,		
Operating Experience of the Nuclear Safety Information Center March 1963 - March 1965 Cottrell 05/07/1965 ORNL without approval  The Determination of Neutron Flux in Nuclear Reactor by the Uncollided-Flux Estimator Reactor by the Uncollided-Flux Estimator Source Applied to Monte Carlo Collisions J. W. Webster 08/23/1965 ORNL ORNL prepared primarily for internal use at the SOURCE, a Neutron Distribution Routine for the OSR Monte Carlo Code ORNL-TM-1192 OSR Monte Carlo Code ORNL-TM-1195 Fluoride Solution Enriched to 5% in 235 U Johnson O7/23/1965 ORNL ORNL prepared primarily for internal use at the Fluoride Solution Enriched to 5% in 235 U Johnson, R. K. Reedy Fig. B. Johnson, R. K. Reedy	524	ORNL-TM-1125	Uranium-Molybdenum Critical Cylinders	J. T. Mihalczo	04/30/1965	ORNL	without approval	reactor safety	good	N	
S25 ORNL-TM-1136 Information Center March 1963 - March 1965 Cottrell 05/07/1965 ORNL without approval prepared primarily for internal use at the OSRNL-TM-1175 ORNL-TM-1195 Fluoride Solution Enriched to 5% in 235 U Signer Property of the Control of the C							not to be given				
S25 ORNL-TM-1136 Information Center March 1963 - March 1965 Cottrell 05/07/1965 ORNL without approval prepared primarily for internal use at the OSRNL-TM-1175 ORNL-TM-1195 Fluoride Solution Enriched to 5% in 235 U Signer Property of the Control of the C			Operating Experience of the Nuclear Safety	J. R. Buchanan, W. B.			public dissemination		report, original,		Mostly describes activities similar to
Reactor by the Uncollided-Flux Estimator Applied to Monte Carlo Collisions J. W. Webster ORNL DRNL-TM-1175 ORNL DRNL-TM-1192 ORNL-TM-1192 OSR Monte Carlo Code ORNL-TM-1195 OR	525	ORNL-TM-1136	Information Center March 1963 - March 1965	Cottrell	05/07/1965	ORNL	without approval		good	N	
Reactor by the Uncollided-Flux Estimator Applied to Monte Carlo Collisions  J. W. Webster  ORNL  ORNL  I. W. Webster  ORNL  ORNL  I. Prepared primarily for internal use at the ORNL  ORNL  II. Prepared primarily for internal use at the ORNL  ORNL  III. Prepared primarily for internal use at the ORNL  ORNL  III. Prepared primarily for internal use at the ORNL  ORNL  III. Prepared primarily for internal use at the ORNL  ORNL  III. Prepared primarily for internal use at the ORNL			The Determination of Neutron Flux in Nuclear								
SOURCE, a Neutron Distribution Routine for the SOURCE, a Neutron Distribution Routine for the ORNL ORNL ORNL ORNL ORNL ORNL ORNL ORNL			Reactor by the Uncollided-Flux Estimator					computational method/data	report, original,		
SOURCE, a Neutron Distribution Routine for the OSR Monte Carlo Code Morrison, D. Irving O7/08/1965 ORNL ORNL prepared primarily for internal use at the OSR Monte Carlo Code Morrison, D. Irving O7/08/1965 ORNL ORNL prepared primarily for internal use at the Computational method/data (2) good N  Criticality of a Single Unit of Aqueous Uranyl Fluoride Solution Enriched to 5% in <sup>235</sup> U Johnson O7/23/1965 ORNL ORNL experimental criticality data good Y  E. B. Johnson, R. K. Reedy E. B. Johnson, R. K. Reedy For internal use at the for internal use	526	ORNL-TM-1175	1	J. W. Webster	08/23/1965	ORNL				N	
SOURCE, a Neutron Distribution Routine for the OSR Monte Carlo Code  OSR Monte Carlo Code  OSR Monte Carlo Code  OSR Monte Carlo Code  ORNL  OTIticality of a Single Unit of Aqueous Uranyl  Fluoride Solution Enriched to 5% in <sup>235</sup> U  Donnson  ORNL					· · · · · · · · · · · · · · · · · · ·			+	t		
S27 ORNL-TM-1192 OSR Monte Carlo Code Morrison, D. Irving 07/08/1965 ORNL ORNL (2) good N  Criticality of a Single Unit of Aqueous Uranyl 528 ORNL-TM-1195 Fluoride Solution Enriched to 5% in <sup>235</sup> U Johnson 07/23/1965 ORNL ORNL experimental criticality data good Y  E. B. Johnson, R. K. Reedy E. B. Johnson, R. K. Reedy For internal use at the			SOURCE, a Neutron Distribution Routine for the	J. T. Mihalczo. G. W.				computational method/data	report, original		
Criticality of a Single Unit of Aqueous Uranyl 528 ORNL-TM-1195 Fluoride Solution Enriched to 5% in <sup>235</sup> U  Dohnson  Or/23/1965 ORNL  ORNL  prepared primarily for internal use at the experimental criticality data good  prepared primarily for internal use at the experimental criticality data good  Y  E. B. Johnson, R. K. Reedy  E. B. Johnson, R. K. Reedy	527	ORNI-TM-1192	1		07/08/1965	ORNI			1	N	
Criticality of a Single Unit of Aqueous Uranyl 528 ORNL-TM-1195 Fluoride Solution Enriched to 5% in 235U  Dohnson  ORNL  ORNL  Dohnson, R. K. Reedy  E. B. Johnson, R. K. Reedy  E. B. Johnson, R. K. Reedy  Dongstandard Single Unit of Aqueous Uranyl J. W. Webster, E. B.  ORNL  ORNL  ORNL  Dongstandard Single Unit of Aqueous Uranyl Experimental use at the seperimental criticality data good Y  Dongstandard Single Unit of Aqueous Uranyl Experimental use at the seperimental criticality data good Y  E. B. Johnson, R. K. Reedy  E. B. Johnson, R. K. Reedy	341	UVE 11V1 1132	SSA MONICE CUITO COME		37,00,1303	OAIVE		<u></u>	Bood		
528 ORNL-TM-1195 Fluoride Solution Enriched to 5% in <sup>235</sup> U Johnson 07/23/1965 ORNL ORNL experimental criticality data good Y  E. B. Johnson, R. K. Reedy for internal use at the report, original, prepared primarily for internal use at the report, original, prepared primarily for internal use at the report, original, prepared primarily for internal use at the report, original, preport, original, prepared primarily for internal use at the report, or internal			Criticality of a Single Unit of Aqueous Uranyl	I W Webster F B					report original		
prepared primarily E. B. Johnson, R. K. Reedy for internal use at the report, original,	F20	ODNI TNA 1105			07/22/1065	ODNI		1	1	v	
E. B. Johnson, R. K. Reedy for internal use at the report, original,	528	OKINE-1IVI-1195	riuoriue Solution Enriched to 5% in "U	JUHISON	0//23/1905	UKNL		experimental criticality data	good		
				50.1							
529   ORNL-TM-1207   Critical Experiments with SPERT-D Fuel Elements   Jr.   07/14/1965   ORNL   ORNL   experimental criticality data   good   Y									1		
	529	ORNL-TM-1207	Critical Experiments with SPERT-D Fuel Elements	Jr.	07/14/1965	ORNL	ORNL	experimental criticality data	good	Y	

	T	Static and Dynamic Transport Calculations for	1			prepared primarily	1			
		Pulsed-Neutron Experiments with Spheres of				for internal use at the	computational method/data	report, original,		
530	ORNL-TM-1213	Uranyl Nitrate Solution	D. W. Magnuson	04/26/1966	ORNL	ORNL	(2)	good	N	
		,		3 1, 22, 2222		not to be given	(-)	8		
		Multiplication Factor of Uranium Metal by One-				public dissemination	computational method/data	report, original,		
531	ORNL-TM-1220	Velocity Monte Carlo Calculations	J. T. Mihalczo	08/19/1965	ORNL	without approval	(2)	good	N	
				30, 20, 2000		prepared primarily	, -,	8		
		SPCTRM - An O5R Monte Carlo Analysis Routine	G. W. Morrison, J. T.			for internal use at the	computational method/data	report, original,		
532	ORNL-TM-1245	for Calculating the Neutron Energy Spectrum	Mihalczo, D. C. Irving	08/26/1965	ORNL	ORNL	(2)	good	N	
		O5R Monte Carlo Analysis Routines for				prepared primarily	· · · · · · · · · · · · · · · · · · ·	Ĭ		
		Calculating Various Mean Times in Reactor	D. C. Irving, G. W.			for internal use at the	computational method/data	report, original,		
533	ORNL-TM-1246	Problems	Morrison, J. T. Mihalczo	02/15/1966	ORNL	ORNL	(2)	good	N	
						prepared primarily	<u> </u>	ř – –		
		ORNL Facilities Capable of Contributing to				for internal use at the		report, original,		Contains several photos and drawings of
534	ORNL-TM-1268	Plutonium Fuel Cycle Development	J. W. Ullman	11/09/1965	ORNL	ORNL		good	N	equipment, facilities, and fuel forms.
						prepared primarily		Ī		
		A Thesaurus of Keywords on Nuclear Fuel	W. H. Bridges, R. L.			for internal use at the		report, original,		
535	ORNL-TM-1285	Technology	Pilloton	10/08/1965	ORNL	ORNL		good	N	
		REACT and CONVRG FORTRAN Subroutines for				prepared primarily				
		Determining Source Convergence for the O5R	G. W. Morrison, J. T.			for internal use at the	computational method/data	report, original,		
536	ORNL-TM-1325	Monte Carlo Neutron Transport Code	Mihalczo, D. C. Irving	02/15/1966	ORNL	ORNL	(2)	good	N	
		3.363.3333	,			not to be given		1		
		The Role of Information Centers; Evaluation of				public dissemination		report, original,		
537	ORNL-TM-1339	Their Effectiveness	F. Kertesz	11/17/1965	ORNL	without approval		good	N	
						prepared primarily	nuclear measurement/data,			
		Point-Set Representation of <sup>238</sup> U Cross Sections;				for internal use at the	computational method/data	report, original,		
538	ORNL-TM-1448	Values and a Fortran Program for Computation	J. W. Webster	06/13/1966	ORNL	ORNL	(2)	good	N	
	0	Taldes and a Fortian Flogram for computation	3. VI. VI. DSC.	00/13/1300			experimental criticality data,	Boon		
		Experimental Determination of Safe Handling				prepared primarily	stainlees steel & cadmium			
		Procedures for High Flux Isotope Reactor Fuel				for internal use at the	inserts, lead, steel and	report, original,		
539	ORNL-TM-1488	Elements Outside the Reactor	S. J. Raffetry, J. T. Thomas	07/1966	ORNL	ORNL	concreterReflection.	good	J-5700	
333	ONNE-TWI-1400	Lientents Outside the Reactor	3. J. Nametry, J. T. Infomas	07/1300	ONIVE	prepared primarily	concrete menection.	good	3-3700	
		Criticality of Lattices of Heat Transfer Reactor	E. B. Johnson, R. K. Reedy			for internal use at the		report, original,		
540	ORNL-TM-1566	Experiment Fuel Elements	Jr.	07/20/1966	ORNL	ORNL	experimental criticality data	good	Υ	
340	OKIVE-TIVI-1300	Experiment ruer Elements	J1.	07/20/1300	ONIVE	prepared primarily	experimental criticality data	good	·	
		ACTIVT - An O5R Monte Carlo Analysis Routine	G. W. Morrison, J. T.			for internal use at the	computational method/data	report, original,		
541	ORNL-TM-1674	for Calculation of Activation	Mihalczo, D. C. Irving	10/03/1966	ORNL	ORNL	(2)	good	N	
341	OMINE-TIVI-1074	Tor Calculation of Activation	Williaiczo, D. C. II Villg	10/03/1300	ONIVE	prepared primarily	(2)	good		Describes early efforts for computer
		Key Word Index Science and Technology				for internal use at the		report, original,		translation from one language to
542	ORNL-TM-1690	Bibliographies January 1965 - June 1966	J. M. Bobb	01/1967	ORNL	ORNL		good	N	another.
342	OKINE-TIVI-1090	bibliographies January 1903 - June 1900	J. IVI. BODD	01/1307	ORIVE	prepared primarily		good	IN	another.
						for internal use at the		report, original,		
543	ORNL-TM-1723	BABEL - Or Translation by Man and Machine	F. Kertesz	01/1967	ORNL	ORNL		good	N	
343	ORINE-TIVI-1725	Prompt Neutron Decay and Reactivity	r. Kertesz	01/1907	ORIVE	prepared primarily		good	IN	From 4 t0 11 each inner & outer
		Measurements in Subcritical Uranium Metal				for internal use at the		report, original,		assemblies in linear, square & triangular
F44	ODAU TA4 4726		I T Milestone	02/1000	ODNII			1	.,	
544	ORNL-TM-1736	Cylinders	J. T. Mihalczo	02/1968	ORNL	ORNL Inner and Outer HIFR	experimental criticality data	good	Y	pitched arrays.
						Fuel Assemblies made				
1						critical with the				
		Critical Lattices of High flow leatons Brosses Fred				Thomas & Raffety	experimental criticality data,			
545	ORNL-TM-1808	Critical Lattices of High flux IsotopeRreactor Fuel	E B Johnson	02/20/1067	ORNL		HIFR Fuel	report, copy,	1 5700	
343	OUNT-1M-1909	elements O5R Monte Carlo Calculations of Low-Enriched	E. B. Johnson	03/20/1967	URINL	8-55).	mirk ruei	good	J-5700	
		Uranium Thermal Critical Assemblies, and				prepared primarily				
		1				for internal use at the	computational method,	report, original,		
546	ORNL-TM-2187	Evidence in Errors in <sup>238</sup> U Cross Sections	J. W. Webster	11/13/1968	ORNL	ORNL	nuclear data	good	N	
						prepared primarily		1		
						for internal use at the		report, original,		
547	ORNL-TM-2200	Estimating the Reliability of Systems	A. P. Fraas	05/1968	ORNL	ORNL	reactor safety	good	N	<u></u>
										Very interesting report addressing actual
						prepared primarily		1		or slang names given to technical terms,
l		Static and Dynamic Measurements with the				for internal use at the		report, original,		processing sites, critical experiment
548	ORNL-TM-2330	Army Pulse Radiation Facility Reactor	J. T. Mihalczo	06/1968	ORNL	ORNL	experimental criticality data	good	Υ	machines, etc.

		<del></del>					T		1	Outlines sections of a SARP (safety
										analysis report for packaging) and
						prepared primarily				provided guidances for the analysis and
						for internal use at the		report, original,		design of fissile and radioactive materials
549	ORNL-TM-2367	The Language of Nuclear Science	F. Kertesz	09/17/1968	ORNL	ORNL		good	N	_
549	URINL-1 IVI-2367	The Language of Nuclear Science	r. Kertesz	09/17/1908	ORNL			good	IN	transport packages.
						prepared primarily	,			
						for internal use at the	equipment/process design,	report, original,		
550	ORNL-TM-2410	Irradiated Fuel Shipping Cask Design Guide	L. B. Shappert	01/29/1969	ORNL	ORNL	transport safety analysis	good	N	
						prepared primarily				
			T. B. Fowler, D. R. Vondy,			for internal use at the	computational method/data	report, original,		
551	ORNL-TM-2496, Rev. 2	Nuclear Reactor Core Analysis Code: CITATION	G. W. Cunningham	07/1969	ORNL	ORNL	(2)	good	N	
						prepared primarily				
		XSDRN: A Discrete Ordinates Spectral Averaging	N. M. Greene, C. W.			for internal use at the	computational method/data	report, original,		
552	ORNL-TM-2500	Code	Craven Jr.	07/1969	ORNL	ORNL	(2)	good	Y	
						prepared primarily				
		The INDEX Data System: An Index of Nuclear	J. L. Lucius, J. D. Jenkins, R.			for internal use at the		report, original,		Crume's PhD thesis regarding fusion
553	ORNL-TM-3334	Data Libraries Available at ORNL	Q. Wright	03/29/1971	ORNL	ORNL	handbook/bibliography	good	N	processes.
		Nonlinear Evolution of Flute-Like Plasma						report, original,		
554	ORNL-TM-3812	Microinstabilities	E. C. Crume Jr.	05/1972	ORNL	[None]		good	N	
· · · · · · · · · · · · · · · · · · ·		Technical Specifications Tower Shielding					operating	report, original,	·	
555	ORNL-TM-4641	Reactor II	Staff	02/1979	ORNL	[None]	procedures/requirements	good	N	
	OWAF-UAL-AOAT	Technical Specifications Tower Shielding	Juli	02/13/3	ONNE	[HOHE]	·	report, original,		
	ODNI TNA ACAA /DA	-	Chaff Can TM 04 40 hal	04/1003	OPM	[None]	operating	1	1	
556	ORNL-TM-4641/R1	Reactor II	Staff-See TM-8140 below	04/1982	ORNL	[None]	procedures/requirements	good	N	
		Technical Specifications Health Physics	-				operating	report, original,	1	
557	ORNL-TM-4637	Research Reactor	Staff	02/1979	ORNL	[None]	procedures/requirements	good	N	
		Technical Specifications Health Physics					operating	report, original,		
558	ORNL-TM-4637/R1	Research Reactor	Staff	11/1985	ORNL	[None]	procedures/requirements	good	N	
								T-		
		Technical Specifications Tower Shielding Reactor					operating	CSIRC/Electroni	T-CSIRC/Vol-	
559	ORNL-TM-8140	ļ <sub>II</sub> .	R. G. Nicols,et al	May, 1982	ORNL	[None]	procedures/requirements	c	3B	
				,,		,				Effort to determine the enrichment of
										unmoderated U metal required for
			J. E. Brolley, F. J. Byerly, B.							criticality. Used 34 tons of natural U
		Neutron Multiplication in a Mass of Uranium	Feld, A. E. Olds, R.							
		-				6				metal stacked on top of the ORNL
		Metal Problem Assignment 307-X10P "SNELL"	Schalettar, L. Slotin, R. B.			Secret, declassified		report, original,		Graphite Reactor in exponential
560	ORNL-CF-1627	Experiment	Stewart	04/01/1944	ORNL	12/11/1956	experimental criticality data	fair	Y	measurements. Focuses on whether the initial delayed
										critical becomes less reactive or promt
										critical upon stopping of a stirrer motor.
										May be a preliminary look at the physics
		Prediction of Criticality Behavior of a Slurry Upon				Secret, declassified	computational method/data	report, original,		of a critical experiment with UO <sub>3</sub> -H <sub>2</sub> O
561	ORNL-CF-53-1-294	Settling	P. R. Kasten	01/26/1953	ORNL	07/02/1957	(1)	good	N	slurries.
		Preliminary Calculations on Storage of MTR Fuel				Secret, declassified	computational method/data	report, original,	<del> </del>	
FC2	ODNI CE E2 9 122	1	I Drasnor	00/21/1052	ODNI	· ·	1 '	1	1	
562	ORNL-CF-53-8-122	Elements	L. Dresner	08/21/1953	ORNL	11/08/1962	(1)	good	N	
		Measurement of the Fast-Neutron Spectrum of	M. P. Haydon, E. B.			Secret, declassified		report, original,	!	
563	ORNL-CF-53-8-146	the Bulk Shielding Reactor Using Nuclear Plates	Johnson, J. L. Meem	08/31/1953	ORNL	05/22/1957	nuclear measurement/data	good	N	
564	ORNL-CF-53-9-139	FOLDER IS EMPTY							ļ	
565	ORNL-CF-54-12-21	FOLDER IS EMPTY								
										Report consists of two memos, to/from
										D. D. Eisenhower and L. Strauss (AEC
										Chairman), and an agreement signed by
1										Strauss and UK officials on 15/06/1955.
										Also a transmittal letter of these
		Critical Experiments on Type AC-1 Aircraft	D. V. P. Williams, J. J. Lynn,			Secret, declassified		report, original,		documents to A. M. Weinberg (ORNL
FCC	ODNI CE E4 43 444	1	A. D. Callihan	13/15/1054	OPM		ovnovimental oriticality day	1	1	
566	ORNL-CF-54-12-114	Propulsion Reactor	A. D. Callinan	12/15/1954	ORNL	10/28/1971	experimental criticality data	good	Y	Director).
		Agreement for Cooperation on the Civil Uses of								
I		Atomic Energy Between the Government of the								
		United States of America and the Government of								
		the United Kingdom of Great Britain and						report, original,		
567	ORNL-CF-55-10-86	Northern Ireland		10/24/1955	ORNL	Unclassified		good	N	Similar content as ORNL-CF-55-10-86.
		Agreement for Cooperation on the Civil Uses of								
1		Atomic Energy Between the Government of the								
		United States of America and the Government of						report, original,		
568	ORNL-CF-55-10-108	Belgium		10/26/1955	ORNL	Unclassified		good	N	Similar content as ORNL-CF-55-10-86.
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		Agreement for Cooperation on the Civil Uses of								
		Atomic Energy Between the Government of the								
		United States of America and the Government of						report, original,		
569	ORNL-CF-55-10-121	Canada		10/28/1955	ORNL	Unclassified		good	N	
						Postricted deslactified	computational method/data	roport original		
570	ORNL-CF-53-10-157	A Critical Size Nemogram	E. L. Zimmerman	10/23/1953	ORNL	02/11/1963		report, original,	N	
370	OKINE-CF-33-10-137	A Critical Size Nomogram	E. L. Zillillerillali	10/23/1933	ORIVE	Secret, declassified	(1)	report, original,	IN	
571	ORNL-CF-54-4-30	Fuel Dump Tanks for HRT	S. Visner		ORNL	08/22/1956	experiment plan/design	good	N	
371	OTTAL CT 54 4 30	Reflector Moderated Critical Assembly	J. VISITEI		OTAL	Secret, declassified	experiment plany design	report, original,		
572	ORNL-CF-54-4-53	Experimental Program	D. Scott, B. L. Greenstreet	04/08/1954	ORNL	12/30/1963	experiment plan/design	good	Υ	
573	ORNL-CF-55-5-181	DOCUMENT MISSING FROM FOLDER		- , , , , , , , , , , , , , , , , , , ,				8		
574	ORNL-CF-54-5-170	DOCUMENT MISSING FROM FOLDER								
						Confidential,				
						declassified		report, original,		
575	ORNL-CF-54-6-16	Dump Tanks for HRT Blanket	S. Visner	06/03/1954	ORNL	06/12/1957	experiment plan/design	good	N	
						Secret, declassified		report, original,		
576	ORNL-CF-54-6-131	Slug Processing in Continuous Dissolver	E. O. Nurmi	06/17/1954	ORNL	11/18/1957	operational/test/material data	good	N	
		Reflector-Moderated-Reactor Design Parameter	C. S. Burtnette, M. E.			Secret, declassified		report, original,		
577	ORNL-CF-54-7-5	Study Part I: Effect of Reactor Proportions	LaVerne, C. B. Mills	11/08/1954	ORNL	02/21/1961	experiment plan/design	good	N	
		Boundary Values for the Inner Radius of a					computational method/data	report, original,		
578	ORNL-CF-54-7-64	Cylindrical Annular Reactor	E. L. Zimmerman	07/10/1954	ORNL	Unclassified	(1)	good	N	
						Secret, declassified	computational method/data	report, original,		
579	ORNL-CF-54-8-221	A Report on Critical Dimensions of Cylinders	R. C. Keen	08/31/1954	ORNL	07/02/1957	(1)	good	N	
580	ORNL-CF-54-9-89	DOCUMENT MISSING FROM FOLDER								
581	ORNL-CF-55-4-22	DOCUMENT MISSING FROM FOLDER						ļ		
582	ORNL-CF-55-6-148	DOCUMENT MISSING FROM FOLDER				6		<del> </del>		
500	ODAU CE EE C 40C	Seals and Packing Materials for Molten Fluoride	M. C. Turnell	00/20/4056	ODAU	Secret, declassified		report, original,		
583	ORNL-CF-55-6-196 ORNL-CF-55-11-104	Salts DOCUMENT MISSING FROM FOLDER	W. C. Tunnell	08/29/1956	ORNL	03/26/1962	operational/test/material data	good	N	
584 585	ORNL-CF-55-11-104 ORNL-CF-56-2-63	DOCUMENT MISSING FROM FOLDER  DOCUMENT MISSING FROM FOLDER						<del> </del>		
363	URINL-CF-50-2-03	DOCUMENT MISSING FROM FOLDER				Confidential,				
		Radiation Incident of February 1, 1956, A				declassified		report, original,		
586	ORNL-CF-56-2-105	Preliminary Report	A. D. Callihan	02/15/1956	ORNL	07/02/1957	criticality accident	good	N	
300	ONIVE CI 30 2 103	Tremmary Report	A. D. Califfali	02/13/1330	OTAL	Confidential,	criticality decident	good		
		An Evaluation of Geometrical Effects in Control				declassified		report, original,		
587	ORNL-CF-56-3-9	Rods	D. W. Magnuson	03/02/1956	ORNL	07/03/1957	experimental criticality data	good	N	
588	ORNL-CF-56-3-159	DOCUMENT MISSING FROM FOLDER		<del> </del>			<u> </u>	Ĭ		
							computational method/data	report, original,		Not critical; multiplication was ~ 10 to
589	ORNL-CF-56-3-170	Neutron Self-Shielding of a Plane Absorbing Foil	J. Bengston	03/01/1956	ORNL	Unclassified	(1)	good	N	12.
		The First Assembly of the Small Two-Region				Secret, declassified		report, original,		
590	ORNL-CF-54-7-159	Reflector Moderated Reactor	D. Scott	07/26/1954	ORNL	08/19/1957	experimental criticality data	good	Υ	
										Provides flux distribution measurements
		The Second Assembly of the Small Two-Region				Secret, declassified		report, original,		and other data for the assembly
591	ORNL-CF-54-8-180	Reflector Moderated Reactor	D. Scott	08/26/1954	ORNL	12/17/1957	experimental criticality data	good	Υ	described in ORNL-CF-54-8-180.
		The Second Assembly of the Small Two-Region				Secret, declassified		report, original,		
592	ORNL-CF-54-9-185	Reflector Moderated Reactor (Part II)	D. Scott, R. M. Spencer	09/27/1954	ORNL	05/19/1959	experimental criticality data	good	Υ	
		Reflector Moderated Critical Assembly				Secret, declassified		report, original,		
593	ORNL-CF-54-10-119	Experimental Program - Part II	B. L. Greenstreet	10/19/1954	ORNL	12/30/1963	experimental criticality data	good	Y	
594	ODNI CE E4 11 22	The First Assembly of the Three-Region Reflector	1	11/05/1054	ORNL	Secret, declassified 12/02/1963	ovnorimental criticality data	report, original,	v	
394	ORNL-CF-54-11-33	Moderated Reactor The Second Assembly of the Three-Region	R. M. Spencer R. M. Spencer, B. L.	11/05/1954	URINL	Secret, declassified	experimental criticality data	good report, original,	т	
595	ORNL-CF-54-11-150	Reflector Moderated Reactor	Greenstreet	11/24/1954	ORNL	11/18/1960	experimental criticality data	good	Υ	
333	O1114E-C1-34-11-130	Three-Region Reflector Moderated Critical	OTCC113H CCL	11/24/1334	ONNE	Secret, declassified	experimental criticality data	report, original,		
596	ORNL-CF-54-12-189	Assembly	R. M. Spencer	12/28/1954	ORNL	03/27/1961	experimental criticality data	good	Υ	
	O.114E CI 34-12-103	Three-Region Reflector Moderated Critical		12/20/1334	CINIL	Secret, declassified	experimental criticality udta	report, original,	·	
597	ORNL-CF-55-1-123	Assembly with 1/16 in. Inconel Shells	R. M. Spencer	01/21/1955	ORNL	11/11/1963	experimental criticality data	good	Υ	
		Three-Region Reflector Moderated Critical		,, 1333		,, 55	, sitesanty data	15.22		
		Assembly with End Ducts and 1/8 in. Inconel								
		Core Shells. CA-21-1 Neutron Flux	S. Synder, J. J. Lynn, E. V.			Secret, declassified		report, original,		
598	ORNL-CF-55-10-142	Measurement.	Sandin, D. Scott	10/28/1955	ORNL	12/30/1963	experimental criticality data	good	Υ	
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							<del></del>			
		Three-Region Reflector Moderated Critical								
		Assembly with End Ducts Experimental Results				Secret, declassified		report, original,		
599	ORNL-CF-56-1-96	with CA-22, Enlarged End Duct Modification	Sandin, S. Synder	01/30/1956	ORNL	12/30/1963	experimental criticality data	good	Y	
		Three-Region Reflector Moderated Critical								
		Assembly with End Ducts Experimental Results	D. Scott, J. J. Lynn, E. V.			Secret, declassified		report, original,		
600	ORNL-CF-56-1-97	with CA-23, Enlarged Island Modification	Sandin, S. Synder	01/30/1956	ORNL	08/22/1962	experimental criticality data	good	Υ	
		Three-Region Reflector Moderated Critical								
		Assembly with End Ducts. Experiment Results	D. Scott, J. J. Lynn, E. V.			Secret, declassified		report, original,		
601	ORNL-CF-56-4-128	with CA-21-2. Reduced Concentration.	Sandin, S. Synder	04/12/1956	ORNL	12/30/1963	experimental criticality data	good	Υ	
		Neutron Self-Shielding of a Purely Absorbing					computational method/data	report, original,		
602	ORNL-CF-56-3-180	Cylindrical Rod	J. Bengston	03/06/1956	ORNL	Unclassified	(1)	good	N	
		A Self-Shielding Factor for Uranium in a Teflon				Secret, declassified	computational method/data	report, original,		
603	ORNL-CF-56-3-183	Lattice	J. Bengston	03/19/1956	ORNL	07/03/1957	(1)	good	N	
	<del> </del>			<del></del>		Confidential,	<u> </u>	ļ		
		Preliminary Report of Critical Experiments in				declassified		report, original,		
604	ORNL-CF-56-7-148	Slab Geometry	J. K. Fox, L. W. Gilley	07/30/1956	ORNL	02/20/1963	experimental criticality data	good	Υ	
605	ORNL-CF-56-8-201	DOCUMENT MISSING FROM FOLDER	J. R. FOX, E. VV. OIIICY	07/30/1330	OTTIVE	02/20/1505	experimental criticality data	good	<u>-</u>	
	OKINE-CF-30-6-201	DOCOMENT MISSING PROMI POLDER								Includes papers and presentations by A.
										M. Weinberg and A. D. Callihan.
										Discusses design and operation of
										- '
										several ORNL-developed test reactors
										and supporting critical experiments, e.g.,
		Neutron and Gamma-Ray Attenuation for a								HRT, APPR and ARE. Several
		Fission Source in Water Comparison of Theory					computational method/data	report, original,		photographs and flow diagrams
606	ORNL-CF-57-3-48	with LTSF Measurements	D. R. Otis	03/12/1957	ORNL	Unclassified	(1)	good	N	included.
								report, original,		
607	ORNL-CF-57-6-69	Molten Fluoride Reactors	Staff	06/27/1957	ORNL	Unclassified	experimental criticality data	good	Υ	
608	ORNL-CF-57-11-15	DOCUMENT MISSING FROM FOLDER								
			F. N. Watson, C. E.							
		Manual of Routine and Emergency Operating	Clifford, M. J. Welch, J. L				operating	report, original,		
609	ORNL-CF-57-11-39	Procedures for the Tower Shielding Facility	.Hull, V. R. Cain	11/25/1957	ORNL	Unclassified	procedures/requirements	good	N	
								report, original,		
610	ORNL-CF-58-3-33	Direct Measurement of the Alpha Value for U <sup>233</sup>	P. R. Kasten	03/10/1958	ORNL	Unclassified	nuclear data/measurement	good	N	
		· · · · · · · · · · · · · · · · · · ·				External distribution		l		
		Gamma-Ray and Fast-Neutron Dose Rates in Air				limited to recipients		report, original,		
611	ORNL-CF-58-4-6	as a Function of Distance from the TSF Reactor	F. N. Watson	04/15/1958	ORNL	indicated, unclassified	dosimetry	good	N	
						prepared primarily				
		High Flux Research Reactor Seminar, March 7,				for internal use at the	experiment plan/design,	report, original,		
612	ORNL-CF-58-6-49	1958 (Seminar No. 4)	J. A. Lane	06/13/1958	ORNL	ORNL	reactor safety	good	N	
613	ORNL-CF-58-8-3	DOCUMENT MISSING FROM FOLDER	J. A. Lanc	00/13/1330	OTTIVE	OTIVE	reactor surety	Bood		
614	ORNL-CF-58-8-5	DOCUMENT MISSING FROM FOLDER  DOCUMENT MISSING FROM FOLDER		<del> </del>			<del> </del>	<del> </del>		
615	ORNL-CF-58-8-15	DOCUMENT MISSING FROM FOLDER  DOCUMENT MISSING FROM FOLDER		<del>                                     </del>			-	-		
	ORNL-CF-58-8-15			-				-		
616	UNIVE-CF-36-9-40	DOCUMENT MISSING FROM FOLDER		<del> </del>		not to be sives		<del> </del>		
		Some Theoretical Considerations Concerning the				not to be given	computational	ronout!-!!		
C	ODNII 65 56 46 44	Measurement of Effective Resonance Integrals in	L B	40/02/5050	05***	public dissemination	computational method/data	report, original,		
617	ORNL-CF-58-10-14	the Bulk Shielding Reactor	L. Dresner	10/03/1958	ORNL	without approval	(1)	good	N	
		Health Physics Research Reactor Operating				not to be given				
		Report November 1, 1964 Through April 30,				public dissemination		report, original,		
618	ORNL-CF-65-6-70	1965	L. W. Gilley, L. B. Holland	06/25/1965	ORNL	without approval	operational/test/material data	good	N	
						not to be given				
		Health Physics Research Reactor Operating				public dissemination		report, original,		
619	ORNL-CF-65-11-75	Report May 1, 1965 Through October 31, 1965	L. W. Gilley, L. B. Holland	11/23/1965	ORNL	without approval	operational/test/material data	good	N	
		Health Physics Research Reactor Operating				not to be given				
		Report November 1, 1965 Through April 30,				public dissemination		report, original,		
620	ORNL-CF-66-6-6	1966	L. W. Gilley, L. B. Holland	06/07/1966	ORNL	without approval	operational/test/material data	good	N	
	<b>+</b>			<del> </del>		not to be given				
		Health Physics Research Reactor Operating				public dissemination		report, original,		
621	ORNL-CF-67-1-5	Report May 1, 1966 Through October 31, 1966	L. W. Gilley, L. B. Holland	01/09/1967	ORNL	without approval	operational/test/material data		N	
		Health Physics Research Reactor Operating		32,03,1307	J.111E	not to be given		10000	.,	
		Report November 1, 1966 Through April 30,				public dissemination		report, original,		
622	ODNI CE 67 9 43		L W Gilloy L B Halland	00/22/4067	OPAU		operational/test/meterial date	1	ķ.	
622	ORNL-CF-67-8-42	1967	L. W. Gilley, L. B. Holland	08/22/1967	ORNL	without approval	operational/test/material data	gooa	N	

	T					not to be given	Ţ			Ţ
		Health Physics Deserve Peaster Operating	D. R. Ward, L. W. Gilley, L.			not to be given public dissemination		ronart original		
622	ODNII 65 60 2 26	Health Physics Research Reactor Operating		02/10/1000	ODNII	ľ	1	report, original,		
623	ORNL-CF-68-2-26	Report May 1, 1967 Through October 31, 1967	B. Holland	02/19/1968	ORNL	without approval	operational/test/material data	good	N	
		Health Physics Research Reactor Operating				not to be given				
ca.	00111 05 00 0 00	Report November 1, 1967 Through April 30,	D. R. Ward, L. W. Gilley, L.	05/47/4050		public dissemination	1	report, original,		The operational report for May 1
624	ORNL-CF-68-6-60	1968	B. Holland	06/17/1968	ORNL	without approval	operational/test/material data	good	N	through October 31, 1968 is missing.
		Health Physics Research Reactor Operating				not to be given				
		Report November 1, 1968 Through April 30,	D. R. Ward, F. F. Haywood,			public dissemination		report, original,		
625	ORNL-CF-69-7-3	1969	L. W. Gilley, L. B. Holland	06/17/1968	ORNL	without approval	operational/test/material data	good	N	
						not to be given				
		Health Physics Research Reactor Operating	D. R. Ward, F. F. Haywood,			public dissemination		report, original,		The first date in the report title should
626	ORNL-CF-70-1-44	Report May 1, 1969 Through October 31, 1969	J. W. Poston, L. B. Holland	01/30/1970	ORNL	without approval	operational/test/material data	good	N	be November 1, 1969.
		Health Physics Research Reactor Operating				not to be given				
		Report November 1, 1970 Through April 30,	D. R. Ward, F. F. Haywood,			public dissemination		report, original,		
627	ORNL-CF-70-6-23	1970	J. W. Poston, L. B. Holland	01/30/1970	ORNL	without approval	operational/test/material data	good	N	
						not to be given				
		Health Physics Research Reactor Operating	D. R. Ward, F. F. Haywood,			public dissemination		report, original,		
628	ORNL-CF-70-11-25	Report May 1, 1970 Through October 31, 1970	J. W. Poston, L. B. Holland	11/11/1970	ORNL	without approval	operational/test/material data	good	N	
		Health Physics Research Reactor Operating				not to be given				
		Report November 1, 1970 Through April 30,	D. R. Ward, F. F. Haywood,			public dissemination		report, original,		
629	ORNL-CF-71-8-7	1971	J. W. Poston, L. B. Holland	08/06/1971	ORNL	without approval	operational/test/material data		N	
			, , , , , ,	<del>                                     </del>		not to be given	. , ,	-		
		Health Physics Research Reactor Operating	D. R. Ward, F. F. Haywood,			public dissemination		report, original,		
630	ORNL-CF-71-12-21	Report May 1, 1971 Through October 31, 1971	J. W. Poston, L. B. Holland		ORNL	without approval	operational/test/material data		N	
		Health Physics Research Reactor Operating	oston, E. b. Honand	-2, 25, 15, 1	J.111E	not to be given		0-30	.,	
		Report November 1, 1971 Through April 30,	D. R. Ward, F. F. Haywood,			public dissemination		report, original,		
631	ORNL-CF-72-7-33	1972	J. W. Poston, L. B. Holland	07/28/1972	ORNL	without approval	operational/test/material data		N	
031	OINIVE-CI -72-7-33	1372	J. W. FOSCOII, E. B. HOHand	07/28/1372	ONNE	not to be given	operational/test/material data	good		
		Uselikh Dhamies Deserveh Deserve Oneservias	D D Ward E E Harrisa							
622	ODNII CE 72 4 E4	Health Physics Research Reactor Operating	D. R. Ward, F. F. Haywood,	04/22/4072	ODNII	public dissemination		report, original,		
632	ORNL-CF-73-1-54	Report May 1, 1972 Through October 31, 1972	J. W. Poston, L. B. Holland	01/22/1973	ORNL	without approval	operational/test/material data	good	N	
		Health Physics Research Reactor Operating				not to be given				
		Report November 1, 1972 Through April 30,	D. R. Ward, F. F. Haywood,			public dissemination		report, original,		
633	ORNL-CF-73-10-11	1973	L. B. Holland	10/03/1973	ORNL	without approval	operational/test/material data	good	N	
						not to be given				
		Health Physics Research Reactor Operating	D. R. Ward, F. F. Haywood,			public dissemination		report, original,		
634	ORNL-CF-74-4-38	Report May 1, 1973 Through October 31, 1973	L. B. Holland	04/22/1974	ORNL	without approval	operational/test/material data	good	N	
		Health Physics Research Reactor Operating				not to be given				
		Report November 1, 1973 Through April 30,	D. R. Ward, F. F. Haywood,			public dissemination		report, original,		
635	ORNL-CF-74-6-59	1974	L. B. Holland	06/28/1974	ORNL	without approval	operational/test/material data	good	N	
			D. R. Ward, F. F. Haywood,			not to be given				
		Health Physics Research Reactor Operating	H. W. Dickson, L. B.			public dissemination		report, original,		
636	ORNL-CF-74-12-21	Report May 1, 1974 Through October 31, 1974	Holland	12/19/1974	ORNL	without approval	operational/test/material data	good	N	
		Health Physics Research Reactor Operating				not to be given				
		Report November 1, 1974 Through April 30,	D. R. Ward, F. F. Haywood,			public dissemination		report, original,		
637	ORNL-CF-75-5-20	1975	L. B. Holland	05/30/1975	ORNL	without approval	operational/test/material data	good	N	
						not to be given				
		Health Physics Research Reactor Operating	H. W. Dickson, L. W. Gilley,	.		public dissemination		report, original,		
638	ORNL/CF-76/34	Report May 1, 1975 Through October 31, 1975	L. B. Holland, D. R. Ward	01/16/1976	ORNL	without approval	operational/test/material data		N	
	1	Health Physics Research Reactor Operating	<u> </u>	<del> </del>		not to be given				
		Report November 1, 1975 Through April 30,	H. W. Dickson, L. W. Gilley,	.		public dissemination		report, original,		
639	ORNL/CF-76/271	1976	L. B. Holland, D. R. Ward	07/12/1976	ORNL	without approval	operational/test/material data		N	
	,		,, p	3.,, 25.0		not to be given	, series details	J. 7-	·••	
		Health Physics Research Reactor Operating	H. W. Dickson, L. W. Gilley,			public dissemination		report, original,		
640	ORNL/CF-77/5	Report May 1 Through October 31, 1976	L. B. Holland, D. R. Ward	01/06/1977	ORNL	without approval	operational/test/material data		N	
0-10	J.1142/C1 ///J	Health Physics Research Reactor Operating	E. S. Hohana, D. N. Walu	31,00,1377	OMINE	not to be given	operational, test/material data	p.550	11	
		Report November 1, 1976 Through April 30,	H. W. Dickson, L. W. Gilley,			public dissemination		report, original,		
CAA	ODNI /CE 77/207				OPAU	l'	1			
641	ORNL/CF-77/307	1977	L. B. Holland, D. R. Ward	05/17/1977	ORNL	without approval	operational/test/material data	gooa	N	
						not to be given				
	ODAN (OF 77 )	Health Physics Research Reactor Operating	H. W. Dickson, L. W. Gilley,		05	public dissemination		report, original,		
642	ORNL/CF-77/483	Report May 1 Through October 31, 1977	L. B. Holland, D. R. Ward	12/01/1977	ORNL	without approval	operational/test/material data	good	N	
		Health Physics Research Reactor Operating				not to be given				
		Report November 1, 1977 Through April 30,	H. W. Dickson, L. W. Gilley,			public dissemination	1	report, original,		
643	ORNL/CF-78/215	1978	L. B. Holland	06/05/1978	ORNL	without approval	operational/test/material data	good	N	

		Т					1			
		Usalik Dharia Dasarah Dasata Osaratira	II W Dishasa I W Cillan			not to be given				
		Health Physics Research Reactor Operating	H. W. Dickson, L. W. Gilley,			public dissemination		report, original,		
644	ORNL/CF-79/14	Report May 1 Through October 31, 1978	L. B. Holland, C. S. Sims	01/12/1979	ORNL	without approval	operational/test/material data	good	N	
		Health Physics Research Reactor Operating				not to be given				
		Report November 1, 1978 Through April 30,				public dissemination		report, original,		
645	ORNL/CF-79/228	1979	L. W. Gilley, L. B. Holland	06/15/1979	ORNL	without approval	operational/test/material data	good	N	
						not to be given				
		Health Physics Research Reactor Operating				public dissemination		report, original,		
646	ORNL/CF-80/4	Report May 1, 1979 Through October 31, 1979	L. W. Gilley, L. B. Holland	01/08/1980	ORNL	without approval	operational/test/material data	good	N	
		Health Physics Research Reactor (HPRR)				not to be given				
		Operating Report November 1, 1979 Through				public dissemination		report, original,		
647	ORNL/CF-80/204	April 30, 1980	L. B. Holland, L. W. Gilley	05/30/1980	ORNL	without approval	operational data	good	N	
		Health Physics Research Reactor (HPRR)				not to be given				
		Operating Report May 1 Through October 31,				public dissemination		report, original,		
648	ORNL/CF-80/331	1980	L. W. Gilley, L. B. Holland	12/08/1980	ORNL	without approval	operational/test/material data	1	N	
		Health Physics Research Reactor (HPRR)				not to be given		1		
		Operating Report November 1, 1980 Through				public dissemination		report, original,		
649	ORNL/CF-81/97	April 30, 1981	L. W. Gilley, L. B. Holland	12/1981	ORNL	without approval	operational/test/material data		N	
043	Olive, Cr. 01/37	Health Physics Research Reactor (HPRR)	L. VV. Gilley, L. D. Holland	12/1301	OTITE	not to be given	operationaly testy material data	Bood	.,	
		1				public dissemination		roport original		
CEO	ODAU (CF 04 /227	Operating Report May 1, 1981 Through October	L W Ciller I B Hellerd	42/44/4004	ODNII	ř.		report, original,		
650	ORNL/CF-81/327	31, 1981	L. W. Gilley, L. B. Holland	12/14/1981	ORNL	without approval	operational/test/material data	guoa	N	
		Health Physics Research Reactor Operating				not to be given				
		Report November 1, 1981 Through April 30,				public dissemination		report, original,		
651	ORNL/CF-82/253	1982	L. W. Gilley, L. B. Holland	08/19/1982	ORNL	without approval	operational/test/material data		N	
		Health Physics Research Reactor Operating						report, original,		
652	ORNL/CF-83/31	Report May 1 Through October 31, 1982	L. B. Holland	02/10/1983	ORNL	Internal use only	operational/test/material data	good	N	
		Health Physics Research Reactor Operating								
		Report November 1, 1982 Through April 30,						report, original,		
653	ORNL/CF-83/225	1983	E. G. Bailiff, L. B. Holland	06/24/1983	ORNL	Internal use only	operational/test/material data	good	N	
		Health Physics Research Reactor Operating						report, original,		
654	ORNL/CF-84/31	Report May 1 Through October 31, 1983	E. G. Bailiff, L. B. Holland	01/18/1984	ORNL	Internal use only	operational/test/material data		N	
		Health Physics Research Reactor Operating						-		
		Report November 1, 1983 Through April 30,						report, original,		
655	ORNL/CF-84/363		E. G. Bailiff. L. B. Holland	09/18/1984	ORNL	Internal use only	operational/test/material data		N	
	02, 0. 0.1, 303	Health Physics Research Reactor Operating	E. G. Bullin, E. B. Holland	03/10/130		internal asc only	operational, test, material data	report, original,		The operational report for November 1,
656	ORNL/CF-85/48	Report May 1 Through October 31, 1984	E. G. Bailiff, L. B. Holland	02/26/1985	ORNL	Internal use only	operational/test/material data	1	N	1984 through April 30, 1985 is missing.
030	ONNE/C1-83/48	Report Way 1 Through October 31, 1984	L. G. Ballill, L. B. Holland	02/20/1989	ONNE	internal use only	operational/test/material data	good		Operational reports for November 1,
		Health Physics Research Reactor Operating						ronart original		1985 through April 30, 1986, and May 1
	00411 /05 05 /403	Health Physics Research Reactor Operating	5 0 0 11111 1 1 1	42/24/4005	00111			report, original,		1986 through October 31, 1986 are
657	ORNL/CF-85/493	Report May 1 Through October 31, 1985	E. G. Bailiff, L. B. Holland	12/31/1985	ORNL	Internal use only	operational/test/material data	good	N	missing.
		Health Physics Research Reactor Operating								
		Report November 1, 1986 Through April 30,						report, original,		
658	ORNL/CF-87/234		E. G. Bailiff, L. B. Holland	07/21/1987	ORNL	Internal use only	operational/test/material data	good	N	
		Health Physics Research Reactor Operating						report, original,		
659	ORNL/CF-87/351	Report May 1 Through October 31, 1987	E. G. Bailiff, L. B. Holland	12/02/1987	ORNL	Internal use only	operational/test/material data	good	N	
										Experimental tests of the Aberdeen
										pulse reactor (very similar to the HPRR)
										to determine if modified
		Health Physics Research Reactor Operating								equipment/procedure could assure
		Report November 1, 1987 Through April 30,						report, original,		subcriticality in event of water
660	ORNL/CF-88/218	i .	E. G. Bailiff, L. B. Holland	12/31/1988	ORNL	Internal use only	operational/test/material data	1	N	immersion.
	,,		, 2, 5, 1,0,10,10	,,,	Aberdeen Proving		experimental criticality data,	5.22	·•	
		Critical Experiment in Support of Enhancing			Ground, Aberdeen		experiment safety analysis,	report, copy,		Same topic as above report by Kazi and
661	No report number	1	A LI Kari LI C Dubuc-lii	00/21/1070	MD, US Army	[None]		1	ķ.	' '
661	No report number	Security of Reactor Storage	A. H. Kazi, H. G. Dubyoski	08/21/1978	Aberdeen Proving	[None]	reactor safety experimental criticality data,	good	N	Dubyoski.
	TECOM Decios: 7 CO U.O. CO	Critical Evacriment in Success of Enhanci			_		1 '			
	1	Critical Experiment in Support of Enhancing			Ground, Aberdeen		experiment safety analysis,	report, copy,		
662	001	Reactor Storage	A. H. Kazi, H. G. Dubyoski	08/1979	MD, US Army	[None]	reactor safety	good	N	
						not to be given				
		Poison Rod Requirements for a Solid-Fuel				public dissemination	computational method/data	report, original,		
	ORNL-CF-59-3-7	Leacher Tank	B. E. Prince	03/02/1959	ORNL	without approval	(1), equipment/process design	good	N	
663	OMNE-CI-33-3-7									
663	ONNE-CI-59-5-7	Chemical Feasibility of Homogeneous Neutron				not to be given				
663	ONNE-CI-33-3-7	Chemical Feasibility of Homogeneous Neutron Poisons for Criticality Control in Fuel				not to be given public dissemination	equipment/material data,	report, original,		

			1							
						not to be given public dissemination		report, original,		
665	ORNL-CF-59-5-79	Design Criteria for a Bila Oscillator	L. Dresner	05/25/1959	ORNL	f .	superiment plan/design	1	NI.	
665	UKNL-CF-59-5-79	Design Criteria for a Pile Oscillator	L. Dresner	05/25/1959	OKNL	without approval	experiment plan/design	good	N	Calculations done with an early
						not to be given				'
		A Comparison of Elementary Criticality				not to be given public dissemination	computational method/data	report, original,		computer code version at LANL; same experiments are considered as for ORNL-
ccc	ORNL-CF-59-6-45	1	C. W. Nestor Jr.	06/11/1050	ODNI	f .	1	1	N	· ·
666	ORINL-CF-59-6-45	Calculations with Experimental Results	C. W. Nestor Jr.	06/11/1959	ORNL	without approval not to be given	(1)	good	IN	CF-59-6-45.
		Multigroup Diffusion Theory Calculations for				_	computational method/data	report, original,		
667	OPNIL CE EO 7 66	4	C W Nester Ir	07/21/1050	ODNI	public dissemination		1	N	
667	ORNL-CF-59-7-66	Recent Critical Experiments	C. W. Nestor Jr.	07/21/1959	ORNL	without approval External transmittal	(1)	good	IN	
						authorized. Also,				
						not to be given				
		Multiplication Managements with Highly				_		roport original		
cco	ODNII 65 50 7 07	Multiplication Measurements with Highly	I T Milesless I I I I	07/27/4050	ODNII	public dissemination		report, original,	v	
668	ORNL-CF-59-7-87	Enriched Uranium Metal Slabs	J. T. Mihalczo, J. J. Lynn	07/27/1959	ORNL	without approval	experimental criticality data	good	Υ	
669	ORNL-CF-59-12-30	DOCUMENT MISSING FROM FOLDER								0.14
						Fortament to a consistant				Addresses reprocessing of spent fuel by
						External transmittal				salt dissolution coupled with conversion
						authorized. Also,				of the uranium to hexafluoride form
		Manganese Bath Measurements of η of U <sup>233</sup> and	R. L. Macklin, G.			not to be given				followed by cold-trapping. Contains
		II <sup>235</sup>	deSaussure, J. D. Kington,			public dissemination		report, original,		numerous flow diagrams, drawings, and
670	ORNL-CF-60-2-84	U <sup>233</sup>	W. S. Lyon	03/15/1960	ORNL	without approval	nuclear measurement/data	good	N	equipment photographs.
1						Internal use only.				
						Also, not to be given				
1			R. P. Milford, W. H. Carr			public dissemination				
			Jr., G. I. Cathers, R. W.			without approval				
			Horton, S. Mann, F. W.			Several pages originally				
1			Miles, J. B. Ruch, C. L.			marked as confidential		report, original,		
671	ORNL-CF-60-3-74, Rev. 1	Volatility Pilot Plant Hazards Review	Whitmarsh	07/26/1960	ORNL	but crossed out.	equipment/process design	good	Y	
ŀ						Internal use only.				
ŀ						Also, not to be given	computational method/data			
		The Applicability of Packed Glass Raschig Rings				public dissemination	(1), operational/test/material	report, original,		
672	ORNL-CF-60-3-103	for Nuclear Safety in Large Vessels	J. P. Nichols	03/24/1960	ORNL	without approval	data	good	N	
						External transmittal				
						authorized. Also,				
						not to be given				
l						public dissemination	experimental criticality data,	report, original,		Note in folder stating "CF-60-4-24 W.
673	ORNL-CF-60-4-12	Critical Experiments for Reactor Physics Studies	R. Gwin, D. W. Magnuson	09/16/1960	ORNL	without approval	nuclear measurement/data	good	Υ	Webster has 4-7-66"
674	ORNL-CF-60-4-24	DOCUMENT MISSING FROM FOLDER								
						External transmittal				
						authorized. Also,				
						not to be given		1		
		Use of Silicon Surface-Barrier Counters in Fast-				public dissemination		report, original,		
675	ORNL-CF-60-5-121	Neutron Detection and Spectroscopy	T. A. Love, R. B. Murray	05/31/1960	ORNL	without approval	dosimetry	good	N	
		- P	,			Unclassified. Also,		1 1		
1		Hazards Summary and Safety Procedures for				not to be given				
1		Reactor Controls Plutonium-Beryllium Neutron				public dissemination	operating	report, original,		
676	ORNL-CF-60-6-20	Source	J. L. Kaufman	06/08/1960	ORNL	without approval	procedures/requirements	good	N	
· · · ·				22,00,2300	02	External transmittal	p. 1111ar cs/r cquirements	10-24		
1						authorized. Also,		1		
						not to be given		1		
1		Neutron Thermalization and Diffusion in Pulsed				public dissemination		report, original,		
677	ORNL-CF-60-7-32	Media	S. N. Purohit	07/11/1960	ORNL	without approval	computational method (1)	good	N	
	ONINE-CI -00-7=32	INICAID	J. IV. FUI OIIIL	37/11/1500	ONNE	Confidential,	computational method (1)	5000	IN	-
						declassified		1		
						05/31/1961. Also,				
1		Use of Glass Raschig Ring Packing in Nuclear				not to be given				
		Safety Control - Report of Trip to Rocky Flats	D E Brookshank I D			_		roport crisinal		
670	ODNII CE CO O EO		R. E. Brooksbank, J. P.	00/10/1000	OPMI	public dissemination	anarational/tart/tari-!	report, original,		
678	ORNL-CF-60-8-58	Plant	Nichols	08/19/1960	ORNL	without approval	operational/test/material data	good	N	

			1			le		TT		
						External transmittal				
						authorized. Also,				
						not to be given				
		Neutron Multiplication Experiments with				public dissemination		report, copy,		
679	ORNL-CF-61-4-33	Enriched Uranium Metal in Slab Geometry	J. T. Mihalczo, J. J. Lynn	04/10/1961	ORNL	without approval	experimental criticality data	fair	Υ	
		The Laboratory Director's Review Committees at				External transmittal		report, original,		
680	ORNL-CF-61-5-90	ORNL	F. Kertesz	05/10/1961	ORNL	authorized.	safety review	good	N	
						External transmittal		8		
						authorized. Also,				
							avacrimental criticality data			
						not to be given	experimental criticality data,			
		Oak Ridge National Laboratory Fast Burst	W. E. Kinney, J. T.			public dissemination	computational method/data	report, copy,		
681	ORNL-CF-61-8-71	Reactor: Critical Experiments and Calculations	Mihalczo	08/24/1961	ORNL	without approval	(1)	fair	Υ	
682	ORNL-CF-61-9-52	DOCUMENT MISSING FROM FOLDER								
										Evaluates samples of the aluminum alloy
										being used at the time for HFIR fuel
										elements. Gallium at 4 times the
						Internal use only.				specification limits was found, but the
		Limiting Critical Concentrations of Nitrate				Also, not to be given				level presented only a small contribution
						public dissemination	computational method/data	report, original,		to the macroscopic absorption cross
683	ORNL-CF-62-10-55	Solutions of Enriched Uranium and U <sup>235</sup> -Th <sup>232</sup>	J. P. Nichols	10/12/1962	ORNL	without approval	(1)	good	N	section.
		Analytical Results and Macroscopic Thermal				Distribution limited to				
		Neutron Absorption Cross-Section Values for				recipients only. For		report, original,		
684	ORNL-CF-63-2-5	Impurities in Type 6061 Al	R. J. Beaver	02/04/1963	ORNL	internal use only.	operational/test/material data	1	N	
		7, 11, 11, 11, 11, 11, 11, 11, 11, 11, 1		, , ,			1,111	1		
						Internal use only.				
						,				
						Also, not to be given				
		Limiting Critical Concentrations of Aqueous				public dissemination	computational method/data	report, original,		
685	ORNL-CF-63-6-56	Nitrate Solutions of Fissile and Fertile Isotopes	J. P. Nichols	06/25/1963	ORNL	without approval	(1)	good	N	
						Distribution limited to				
		Report on a Recent Conference on Scientific				recipients only. For		report, original,		
686	ORNL-CF-64-9-21	Information Problems, Number 2	F. Kertesz	09/11/1964	ORNL	internal use only.		good	N	
								ļ		
						Internal use only.				
						Also, not to be given				
						public dissemination	computational method/data	report, original,		
687	ORNL-CF-63-9-49	Concrete Shield Calculations	D. K. Trubey	09/20/1963	ORNL	without approval	(1)	good	N	
			İ							
						Internal use only.				
		Operating Safety Limits for the Oak Ridge	J. D. Kington, F. C.			Also, not to be given		1		
		National Laboratory Bulk Shielding Facility	Maienschein, C. C.			public dissemination	operating	report, original,		
688	ORNL-CF-64-10-2	Reactors (BSR-I, BSR-II, and PCA)	Webster	10/02/1964	ORNL	without approval	procedures/requirements	good	N	
	OIIIVE CI 04 10 2	neuctors (BSN 1, BSN 11, and 1 CA)	Webster	10,02,1304	OTATE	Distribution limited to	procedures/requirements	Bood		
		1064 PORC Pavious of the Tower Chiefding			ORNL, Union Carbide			ronart cons		
		1964 RORC Review of the Tower Shielding						report, copy,		
689	ORNL-CF-65-2-17	Facility	Staff	02/10/1965	Corporation	internal use only.		fair	N	
						Internal use only.				
						Also, not to be given				
		Tower Shielding Reactor II Operating Report				public dissemination		report, original,		
690	ORNL-CF-65-4-3	November 1, 1963 Through October 31, 1964	L. B. Holland	04/01/1965	ORNL	without approval	operational/test/material data		N	
	+			2., 22, 2333			The state of the s	15.22		
						Internal use only				
						Internal use only.				
						Also, not to be given				
		Criticality Safety Tests for a Proposed Irradiation				public dissemination		report, original,		MPRE - Medium-Power Reactor
691	ORNL-CF-65-6-72	Facility	E. B. Johnson	06/30/1965	ORNL	without approval	experimental criticality data	good	N	Experiment
						Internal use only.		1		
		Summary of Reports and Technical Papers That				Also, not to be given		1		
		Have Been Issued or Are Being Prepared in				public dissemination		report, original,		
603	ODNI CE CE 40 0		A D Frons	10/11/1005	000	IF	bandbaak/bibli	1		
692	ORNL-CF-65-10-9	Connection with the MPRE Program	A. P. Fraas	10/11/1965	ORNL	without approval	handbook/bibliography	good	N	
						Internal use only.				
						Also, not to be given		1		
		Tower Shielding Reactor II Operating Report May	J. L. Hull, L. B. Holland, J. J.			public dissemination		report, original,		Duplicate report copy/folder (listed
693	ORNL-CF-65-11-49	1, 1965 Through October 31, 1965	Manning	11/23/1965	ORNL	without approval	operational/test/material data	good	N	above)
		[ , 0		, ,,=====	1		, ,	<u>  </u>		

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		Health Physics Research Reactor Operating				not to be given public dissemination		report, original,		Report on I.A.E.A. Symposium on Criticality Control of Fissile Materials and Visits with French and English Groups Concerned with Criticality. Many details
694	ORNL-CF-65-11-75	Report May 1, 1965 Through October 31, 1965	L. W. Gilley, L. B. Holland	11/23/1965	ORNL	without approval	operational/test/material data		N	of current experimental activities.
								report, original,		
695	ORNL-CF-65-12-42	Report of Foreign Travel	J. T. Thomas	12/13/1965	ORNL	Official Use Only		good	N	
696	ORNL-CF-65-8-55	DOCUMENT MISSING FROM FOLDER								<del></del>
697	ORNL-CF-68-4-70	Preliminary Proposal for Experiments with a Repetitively Pulsed Assembly	J. T. Mihalczo	04/22/1968	ORNL	Internal use only. Also, not to be given public dissemination without approval	experiment plan/design	report, original,	N	
		The Paneticities of High Physicsters Panetas				Internal use only. Also, not to be given		rapart original		
698	ORNL-CF-75-9-13	The Reactivities of High-Flux Isotope Reactor Fuel Elements 148 Through 162, Memo No. 16	E. B. Johnson	09/22/1975	ORNL	public dissemination	experimental criticality data	report, original,	N	
050	UNINE-CF=/J=J=13	Nuclear Criticality Safety Assessment of Oak	L. D. JUIIISUII	03/22/13/3	UNIVL	without approval	facility, process or storage	good report, original,	IN	
699	ORNL/CSD/TM-58	Ridge Research Reactor Fuel Element Storage	J. T. Thomas	06/1978	ORNL	[None]	analysis	good	N	
055	ONNE, CSD, TWI-SO	Mage Research Reactor Fuel Element Storage	J. T. THOMAS	00/1378	ONIVE	[None]	computational method/data	report, original,	- IN	-
700	ORNL/CSD-7	Users Guide to MORSE-SGC	S. K. Fraley	03/1976	ORNL	[None]	(2)	good	N	
	. ,	A 218-Group Neutron Cross-Section Library in	,	,		11	, ,	0		
		the AMPX Master Interface Format for Criticality	W. E. Ford III, C. C.					report, original,		
701	ORNL/CSD/TM-4	Safety Studies	Webster, R. M. Westfall	07/1976	ORNL	[None]	nuclear measurement/data	good	N	
	<del></del>		,	<u> </u>		<u> </u>	operating			
					Oak Ridge Operations		procedures/requirements,			
		Uranium Hexafluoride Handling Procedures &			Office, U. S. Atomic		equipment/material data,	report, original,		
702	ORO-651 Revision 2	Container Criteria		11/1968	Energy Commission	[None]	equipment/process design	good	N	
		An Empirical Correlation of the Experimental					experimental criticality data,			
		Data on Homogeneous Critical Assemblies of			U.K.A.E.A. (Risley,		handbook, computational	report, original,		
703	PG Report 97 (R) 2nd Edition	Uranium and Hydrogen of All Enrichments	B. G. Owen	11/1960	Lancashire)	[None]	method/data (1)	good	N	
			H. Goodfellow, W. B. McCormick, M. H. McTaggart, V. G. Stupart,		Aldermaston A.E.R.E.		experiment plan/design,	report, original,		
704	PR/OP/1	Design Basis for the Aldermaston Pulsed Reactor		02/1964	(U.K.A.E.A.)	Official Use Only	reactor safety	good	N	
705	PTR-359	Design Principle to Ensure Safety in Shipping and Storing Fissile Material	R. W. Thomas, R. L. Doan, J. P. Lyon, J. R. Huffman, F. H. Tingey, J. B. Philipson,	11/24/1958	Philips Petroleum Company and Idaho Operations Office of the U. S. A. E. C.	[None]	criticality safety analysis, design guidance	report, original,	N	
			·							Contains several penciled notes.
706	PWAC-475	Operating Limits and Safety Analysis for Split- Table, Dry, Flexible Critical Experiments at the Canel Nuclear Physics Laboratory	E. L. Paradis, W. F. Welsh, L. E. Kapinos	09/1965	Canel Nuclear Physics Laboratory, Middletown CT, Pratt & Whitney Aircraft		experiment safety analysis	report, original,	N	Maximum observed muliplication was ~ 3. Examined effect of personnel reflection on air-spaced HEU metal arrays.
					Rocky Flats Plant, The		<u> </u>			Many types of materials/configurations
		Neutron Multiplication Measurements of Oralloy			Dow Chemical	Secret, declassified		report, original,		evaluated. Maximum observed
707	RFP-51	Units in Arrays	C. L. Schuske	06/29/1955	Company	05/06/1961	experimental criticality data	good	Υ	muliplication was ~ 2.
708	RFP-58	Industrial Criticality Measurements on Oralloy and Plutonium	C. L. Schuske, M. G. Arthur, D. F. Smith	01/24/1956	Rocky Flats Plant, The Dow Chemical Company	Secret, declassified 03/22/1972	experimental criticality data	report, original,	Y	Includes situation where two planar arrays are separated by a concrete wall.  Maximum observed muliplication was ~ 3 to 4.
700	141 33	Neutron Multiplication Measurements in Parallel	prenar, D. I. Simul	01/24/1930	Rocky Flats Plant, The Dow Chemical		experimental criticality udita	report, original,		Maximum observed muliplication was ~
709	RFP-59	Arrays of Oralloy Units	C. L. Schuske	02/06/1956	Company	08/29/1960	experimental criticality data	good	Υ	10.
		Criticality Measurements on Plutonium Metal	C. L. Schuske, M. G.		Rocky Flats Plant, The Dow Chemical	Secret, declassified		report, original,		Part of the introduction is excerpted.  Maximum observed multiplication was ~
710	RFP-63	Preliminary to the Design of a Melting Crucible	Arthur, D. F. Smith	06/01/1956	Company	06/21/1963	experimental criticality data	good	Y	10.

					T	Secret, downgraded to	1	ŢŢ		Ţ
						Confidential				
						01/28/1964, stamped				
					Pocky Flats Plant The	as declassifed but final				
		Noutron Multiplication Measurements on	C. L. Schuske, M. G.		Dow Chemical	declassification date		roport conv		Maximum observed multiplication was ~
711	DED CC	Neutron Multiplication Measurements on		09/06/1056			ovnovimental oriticality data	report, copy,	Υ	Maximum observed multiplication was ~
/11	RFP-66	Oralloy Slabs Immersed in Solutions	Arthur, D. F. Smith	08/06/1956	Company	not indicated.	experimental criticality data	fair	<u>T</u>	15.
										2 to 3. Material was UO <sub>4</sub> xH <sub>2</sub> O with H/U
					Rocky Flats Plant, The					values of ~ 20 to 30, at approximately
		Neutron Multiplication Measurements on	C. L. Schuske, M. G.		Dow Chemical	Secret, declassified		report, original,		
712	RFP-69	Oralloy Slabs Immersed in Solutions Part II	Arthur, D. F. Smith	10/25/1956	Company	07/29/1960	experimental criticality data	good	Υ	90% <sup>235</sup> U enrichment.
										Analyzes operational data for U
										inventory estimates for materials to be
					Rocky Flats Plant, The					salvaged, estimates the probability for a
		In Situ Neutron Multiplication Measurements on			Dow Chemical	Secret, declassified		report, original,		criticality accident, makes
713	RFP-91	a Calcining Furnace	Schuske, D. F. Smith	12/08/1957	Company	08/29/1960	experimental criticality data	good	Υ	recommendations.
					Rocky Flats Plant, The					
			A. Goodwin Jr., A. N.		Dow Chemical	Secret, declassified		report, original,		
714	RFP-104	Operations	Nickel	04/18/1958	Company	06/21/1963	operational/test data	good	N	
		1235			Rocky Flats Plant, The			1		
745	DED 400	Method for Calculating U <sup>235</sup> Metal Storage	C. I. Charalta	05/07/4055	Dow Chemical	Secret, declassified	computational method/data	report, original,		
715	RFP-108	Arrays	C. L. Shuske	05/07/1958	Company	06/21/1963	(1)	good	N	D
										Document does not appear to be a
										formal report, no formal cover page or
										issue date (date indicated is a stamped
										date of receipt). Includes a copy of a
					Rocky Flats Plant, The		experimental criticality data,			memo from Callihan to Goodwin date
			A. Goodwin Jr., C. L	/ /	Dow Chemical	Secret, declassified	computational method/data	report, original,		07/10/1964, transmitting ORNL
716	RFP-123	Plutonium Graphite Assemblies	Schuske	09/29/1958	Company	08/13/1959	(1)	good	Υ	experimental data.
					Rocky Flats Plant, The	2		1		
				07/40/4064	Dow Chemical	fa. 1	computational method/data	document,		
717	No report number	Nuclear Safety of Pyrex Glass Poisoned Systems	A. Goodwin Jr.	07/10/1964	Company	[None]	(1)	copy, good	N	
		As Familian Interpretation of Association	C. I. Calauralia, C. III		Rocky Flats Plant, The					
740	252 440	An Empirical Interpretation of Annuli Critical	C. L. Schuske, G. H.	40/20/4050	Dow Chemical		computational method/data	report, original,		
718	RFP-149	Mass Data	Bidinger	10/26/1959	Company	Unclassified	(1)	good	N	Multiplication experiments.
			A Conductor In C. I		Rocky Flats Plant, The		experimental criticality data,			
710	DED 450	Distantion Complite Assemblies Boot II	A. Goodwin Jr., C. L	00/40/4050	Dow Chemical	Secret, declassified	computational method/data	report, original,	.,	
719	RFP-158	Plutonium Graphite Assemblies Part II	Schuske	08/10/1959	Company	01/14/1960	(1)	good	Y	
		Isolation Thickness of Water for UO <sub>2</sub> F <sub>2</sub> Solution			Rocky Flats Plant, The Dow Chemical	•	computational mathed/data	roport original		
720	DED 160		C I Cabualta A N Niekal	12/21/1050	1	Unalassified	computational method/data	report, original,	N	Multiplication avacriments
720	RFP-169	Systems	C. L. Schuske, A. N. Nickel C. L. Schuske, A. Goodwin	12/21/1959	Company Rocky Flats Plant, The	Unclassified	(1)	good	N	Multiplication experiments.
		Interaction of Two Metal Slabs of Plutonium in	Jr., G. H. Bidinger, D. F.		Dow Chemical			report, original,		Maximum observed multiplication was ~
721	RFP-174	Plexiglas	Smith	12/28/1959		Secret, declassified 03/16/1960	ovnorimental criticality data	good	Υ	10.
	M 1 -1/4	i ichigida	C. L. Schuske, G. H.	12/20/1739	Company Rocky Flats Plant, The		experimental criticality data	5000		100.
			Bidinger, A. Goodwin Jr.,		Dow Chemical	Secret, declassified		report, original,		
722	RFP-178	Plutonium Plexiglas Assemblies	D. F. Smith	01/20/1960	Company	03/16/1960	experimental criticality data	good	Υ	Multiplication experiments.
122	111 170	Tatomani Flexigias Assemblies	D. T. Silliul	01/20/1300	Rocky Flats Plant, The		experimental criticality data,	Bood	·····	indiciplication experiments.
		Criticality Studies of Enriched Uranium Metal in	A. Goodwin Jr., G. H.		Dow Chemical		computational method/data	report, original,		
723	RFP-182	UO <sub>2</sub> (NO <sub>3</sub> ) <sub>2</sub> Solutions	Bidinger, C. L. Schuske	01/28/1960	Company	Unclassified	(1)	good	Υ	Multiplication experiments.
		3321.3312 301410113		,, 1550	Rocky Flats Plant, The		experimental criticality data,		•	
			G. H. Bidinger, C. L.		Dow Chemical		computational method/data	report, original,		
724	RFP-190	Plutonium Plexiglas Assemblies Part II	Schuske, D. F. Smith	07/27/1960	Company	Unclassified	(1)	good	Υ	Multiplication experiments.
	250	Nuclear Safety Experiments on Plutonium and	2235, 21.1.5111111	3.,2.,1300	Rocky Flats Plant, The		experimental criticality data,			
		Enriched Uranium Hydrogen Moderated	G. H. Bidinger, C. L.		Dow Chemical		computational method/data	report, original,		
725	RFP-201	Assemblies Containing Boron	Schuske, D. F. Smith	10/13/1960	Company	Unclassified	(1)	good	Υ	Multiplication experiments.
	202		2235, 2 5111101	10, 10, 100	Company		1-7	10000		General discussion of two subcritical
					Rocky Flats Plant, The		experimental criticality data,	1		measurement techniques - 1/M
			C. L. Schuske, D. F. Smith,		Dow Chemical		computational method/data	report, original,		measurements and pulsed-neutron
726	RFP-213	Plexiglas Reflected Assemblies of Plutonium	C. L. Bell	01/10/1961	Company	Unclassified	(1)	good	Υ	source measurements.
	229	g-as reflected rissembles of Flatorium	2. 2. 50.	32, 20, 2001	Rocky Flats Plant, The		\-',	1000		
					Dow Chemical			report, original,		
727	RFP-245	Criticality Measurements Performed In Situ	C. L. Schuske	11/15/1961	Company	Unclassified	experiment plan/design	good	N	Multiplication experiments.
,,,,	273	personally incusurements remormed in situ	o. z. ochoske	11/13/1301	Company	ociassifica	experiment plantacing	boou	1.8	такарисации схренитена.

		1						1		Multiplication experiments. Folder
										contains four handwritten pages of
					Rocky Flats Plant, The	2	experimental criticality data,			calculations and graphs, several of which
		Nuclear Safety Measurements on Systems	C. L. Schuske, G. H.		Dow Chemical		computational method/data	report, original,		have Schuske's initials and dates in late
728	RFP-246	Containing Boron and Enriched Uranium	Bidinger	10/24/1961	Company	Unclassified	(1)	good	Υ	1961.
		<u> </u>			Rocky Flats Plant, The					
		Industrial Criticality Measurements on Oralloy	C. L. Schuske, C. L. Bell, G.		Dow Chemical			report, original,		
729	RFP-248	and Plutonium Part II	H. Bidinger, D. F. Smith	01/10/1962	Company	Unclassified	experimental criticality data	good	Υ	
	2.0	and ridesmann are n	The Didningery Dr. 11 Strines	01/10/1502	Company	- Circlessines	experimental criticality data	Bood		May not be a formally issued report; no
										report cover or date are included.
					Rocky Flats Plant, The					Contains an inserted note page from
		Empirical Analysis of Critical Mass Data Involving			Dow Chemical		computational method/data	report, original,		"JERE" stating that "Clarence gave me
730	RFP-258	Enriched Uranium Metal in Water	C. L. Schuske, C. L. Bell	03/21/1962	Company	Unclassified	(1)	good	N	this report at the ACS Meeting"
730	NFF-238	Ellitched Oranium Wetai in Water	C. L. SCHUSKE, C. L. BEII	03/21/1902	Rocky Flats Plant, The		(1)	good	IN	this report at the ACS Meeting
					Dow Chemical			roport conv		Folder contains memos to/from Schske
721	RFP-285	Durahility Tosts on Duray Pasship Dings	C. L. Schuske. H. W. King			[None]	anarational/tast/material data	report, copy,	N	
731	KFP-285	Durability Tests on Pyrex Raschig Rings	C. L. Schuske, H. W. King		Company Rocky Flats Plant, The	[None]	operational/test/material data	good	N	and J. T. Thomas.  May not be a formally issued report; no
		Empirical Analysis of Critical Bare Arrays of	C. I. Cabaralas D. D. Farrat			•				
700	252 245		C. L. Schuske, B. B. Ernst,	05/00/4050	Dow Chemical		computational method/data	report, original,		report cover or date are included.
732	RFP-315	Cylinders Containing Enriched UO <sub>2</sub> (NO <sub>3</sub> ) <sub>2</sub>	H. W. King	05/29/1963	Company	Unclassified	(1)	good	N	Contains several handwritten notes
		True Francisco antal Cult Critical Assess of Du (NO.)			Rocky Flats Plant, The	2				
		Two Experimental Sub Critical Arrays of Pu(NO <sub>3</sub> ) <sub>4</sub>			Dow Chemical			report, original,		
733	RFP-325	Solution	C. L. Schuske		Company	[None]	experimental criticality data	good	Υ	
					Rocky Flats Plant, The	2				Data for HEU metal annular
		Manual for the Use of Borosilicate Raschig Rings	W. E. Schunter, R. W.		Dow Chemical		operating	report, original,		spheres/hemispheres for various
734	RFP-329	as a Fixed Nuclear Poison	Woodward	10/03/1963	Company	Unclassified	procedures/requirements	good	N	reflector/internal material conditions.
		Research and Quarterly Progress Report			Rocky Flats Plant, The	2				
		November, December, 1966, and January 1967			Dow Chemical			report, original,		
735	RFP-898-A	Nuclear Safety	C. L. Schuske	02/23/1967	Company	Official Use Only	experimental criticality data	good	Υ	
		Calculated Critical Radii of Spheres of Plutonium			Rocky Flats Plant, The	2				
		239 and Uranium 233 with Various Spherical	D. R. Ferguson, D. C.		Dow Chemical		computational method/data	report, original,		
736	RFP-936	Reflectors	Coonfield	06/09/1967	Company	[None]	(2)	good	N	
		Empirical Analysis of Spherical and			Rocky Flats Plant, The			1		Data for HEU metal annular spheres for
		Hemispherical Assemblies of Enriched Uranium	B. B. Ernst, C. L. Schuske,		Dow Chemical		computational method/data	report, original,		various reflector/internal material
737	RFP-939	Metal	H. W. King	06/09/1967	Company	[None]	(1)	good	N	conditions.
			,	, ,	Rocky Flats Plant, The					
		Research and Quarterly Progress Report			Dow Chemical			report, original,		Mathematical routine for empirical fits,
738	RFP-956	February, March, and April 1967 Nuclear Safety	C. L. Schuske	02/23/1967	Company	Official Use Only	experimental criticality data	good	Υ	Fortran programming.
		, , , , , , , , , , , , , , , , , , , ,		, , , , , , , , , , , , , , , , , , , ,	Rocky Flats Plant, The	<del></del>				
			H. E. Clark, D. C. Coonfield,		Dow Chemical		computational method/data	report, original,		
739	RFP-968	Polynomial Curve Fit by Least Squares	L. E. Jackson	07/11/1967	Company	[None]	(3)	good	N	
				0., -, -, -	Rocky Flats Plant, The		experiment safety analysis,	8		
		Plutonium Handling Safety Analysis of the Rocky			Dow Chemical		experiment plans, operating	report, original,		
740	RFP-977	Flats Nuclear Safety Facility	D. C. Hunt, G. Tuck	11/08/1967	Company	[None]	procedures/requirements	good	N	
740	3//	Critical Masses of Oil Reflected, Enriched	S. C. Hulle, G. Fuck	11,00,1307	Rocky Flats Plant, The		experimental criticality data,	Bood	- 14	
		Uranium Metal Assemblies with Polyurethane			Dow Chemical		computational method/data	report, original,		
741	RFP-1017	Centers	B. B. Ernst	09/06/1967	Company	[None]	(2)	good	Υ	
/+1	M 1 -1017	- Centers	D. D. LITISE	03/00/130/	Rocky Flats Plant, The		(-)	БООЙ		
		The Effects of Spatial Resolution on Critical Mass			Dow Chemical		computational method/data	report original		
742	RFP-1133	The Effects of Spatial Resolution on Critical Mass Calculations	D. C. Coonfield, D. C. Hunt	05/31/1968	Company	[None]	(2)	report, original,	N	
742	IVI F =1133	Calculations	D. C. COOIIIIEIU, D. C. HUNT	03/31/1908	Rocky Flats Plant, The		\ <u>^</u> 1	good	ıN	-
							computational mathed/d-t-	roport original		
742	DED 1134	Callinian Brobability Criticality Calculation	D. C. Humt	04/16/1000	Dow Chemical	[None]	computational method/data	report, original,		
743	RFP-1134	Collision Probability Criticality Calculations	D. C. Hunt	04/16/1969	Company	[None]	(2)	good	N	
		Callisian Brobability Calculations of			Rocky Flats Plant, The	1	computational method/data	roport origin-!		
744	DED 1106	Collision Probability Calculations of	D. C. Humt	01/10/1000	Dow Chemical	[None]	1 '	report, original,		
744	RFP-1186	Multiplication Factors for Lattices	D. C. Hunt	01/10/1969	Company	[None]	(2)	good	N	
		Cutting lite . Down and a set of the set	D B 5		Rocky Flats Plant, The	:		l		
	252 4206	Criticality Parameters Affecting Equipment	D. R. Ferguson, C. Lee	00/05/:55	Dow Chemical	fa. 3	computational method/data	report, original,		
745	RFP-1206	Design for Fluoride Volatility Process	Schuske	08/29/1968	Company	[None]	(2), equipment/process design	good	N	Maximum multiplication ~ 20
					Rocky Flats Plant, The	2		1		
		Neutron Multiplication Measurements of			Dow Chemical			report, original,		
746	RFP-1242	Plutonium Ingots in Arrays	D. R. Ferguson	12/15/1968	Company	[None]	experimental criticality data	good	Υ	
					Rocky Flats Plant, The	2	experimental criticality data,	1		
		Critical Parameters of a Uranium Solution Slab-			Dow Chemical		computational method/data	report, original,		
747	RFP-1314	Cylinder System	G. Tuck, H. E. Clark	12/16/1970	Company	[None]	(2)	good	Υ	1

March   Marc		T	The Effects of Energy Self-Shielding in Reflected	T		Rocky Flats Plant, The		1	1		
Part   1975   Contact			1					computational method/data	report original		
Part	748	REP-1327	1	D C Hunt D C Coonfield	12/29/1969		[None]		1	N	
Note   Marches	740	101 1327	Calculations	D. C. Hune, D. C. coomicia	12/23/1303			(2)	good		
Price   Pric			Mixtures of Plutonium and Enriched Uranium in	C. L. Schuske, D. C.		1		computational method/data	report, original.		
No.   Part   1906   Part   1	749	RFP-1489	1	· ·	08/10/1971		[None]		1	N	
S.   A.   Note   S.   Note		111 2103	Jour Scomeny	Coomeia	00/10/13/1			(-)	Bood		
March   Marc				S. J. Altschuler, C. L.				computational method/data	report, original.		
Procedure   Proc	750	RFP-1585	Models for the Safe Storage of Fissile Metal		09/24/1971		[None]			N	
1975   1975									<u> </u>		
1975   1975			Criticality Calculations for Plutonium Nitrate					computational method/data	report, original,		
Part	751	RFP-2034	1	D. Dickinson	05/09/1973	Company	[None]	(2)	1	N	
2012-09F						Rocky Flats Plant, The	!				
The RAND State of Sta						Dow Chemical			report, original,		
Composition	752	2012-68PE	RFD Container Model 55/30	F. E. Adcock	09/1968	Company	[None]	transport safety analysis	good	N	
175						The RAND					
The Criticality and Some Potentialities of "Critical Part Some Potentiality			A Brief Description of Some RAND Reactor			Corporation, Santa	Secret, declassified	computational method/data	report, original,		
The NAMO Composition of the Criticality and Some Potentialities of "Carly Mexicos" Rectors" Rectors	753	RM-847	Studies	G. Safonov	05/21/1952	Monica CA	07/06/1960	(1)	good	N	
The Critically and Some Potentialities of Young Section (Comparation and Section Secti											Does not contain data; describes input
Mail Section   Mail						The RAND					1.7
Sanda National Laboratory, sendig Corporation  75 SCR 13-60 Sinvironmental Data Bank R. Corn Jr. 12/1960 Corporation  8 L Coats, P. D. O'Brien O'1/1967 Corporation  8 L Coats, P. D. O'Brien O'1/1967 Corporation  8 L Coats, P. D. O'Brien O'1/1967 Corporation  75 SCR 8-6-2706 SPR I Safety Analysis Report R. L Coats, P. D. O'Brien O'1/1967 Corporation  75 SCR 8-6-2706 SPR I Safety Analysis Report R. L Coats, P. D. O'Brien O'1/1967 Corporation  75 SCR 8-6-2706 SPR I Safety Analysis Report R. L Coats, P. D. O'Brien O'1/1967 Corporation  75 SCR 8-6-2706 SPR I Safety Analysis Report R. L Coats, P. D. O'Brien O'1/1967 Corporation  75 SCR 8-6-2706 SPR I Safety Analysis Report R. L Coats, P. D. O'Brien O'1/1967 Corporation  75 SCR 8-6-2706 SPR I Safety Analysis Report R. L Coats, O'5/1970 Corporation  75 SCR 8-6-2706 SPR I Safety Analysis Report R. L Coats, O'5/1970 Corporation  75 SCR 8-6-2706 SPR I Safety Analysis Report R. L Coats, O'5/1970 Corporation  75 SCR 8-6-2706 SPR I Safety Analysis Report R. L Coats, O'5/1970 Corporation  75 SCR 8-6-2706 SPR I Safety Analysis Report R. L Coats, O'5/1970 Corporation  75 SCR 8-6-2706 SPR I Safety Analysis Report R. L Coats, O'5/1970 Corporation  75 SCR 8-6-2706 Potential EDNA Performance Characteristics  8 L Coats O'5/1970 Sendia National Laboratory, Sendia Corporation  75 SCR 8-6-7-0-64 Potential EDNA Performance Characteristics  8 L Coats O'5/1970 Sendia National Laboratory, Sendia Corporation  75 SCR 8-6-7-0-64 Potential EDNA Performance Characteristics  8 L Coats O'5/1970 Sendia National Laboratory, Sendia Corporation  75 SCR 8-6-7-0-64 Potential EDNA Performance Characteristics  8 L Coats O'5/1970 Sendia National Laboratory, Sendia Corporation  75 SCR 8-6-7-0-64 Potential EDNA Performance Characteristics  8 L Coats O'5/1970 Sendia National Laboratory, Sendia Corporation  75 SCR 8-6-7-0-64 Potential EDNA Performance Characteristics  8 L Coats O'5/1970 Sendia National Laboratory, Sendia Corporation  75 SCR 8-6-7-0-64 Potential EDNA Performance Characteristics  8 L Coats O'5/			The Criticality and Some Potentialities of "Cavity			Corporation, Santa	· ·	computational method/data	report, original,		primarily intended for deployed
Sandia National Laboratory, Sandia National National National National National Nation	754	RM-1520	Reactors"	G. Safonov	07/17/1955	Monica CA		(1)	good	N	weapons components.
Laboratory, Sandla Spania Property organia Property organ											
## Corporation   Property   Prope						Sandia National					
Fig. 1. Coats SPR II Safety Analysis Report R. L. Coats, P. D. O'Brien O1/1967 Corporation Sandia National Laboratory, Sandia National Nation						1					
Set May 1 Set Ma	755	SCDR 312-60	Environmental Data Bank	R. Corn Jr.	12/1960	Corporation	written permission	operational/test/material data	good	N	
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Acceptance   Scare											
SC-RR-66-2706 SPR II Safety Analysis Report R. L. Coats, P. D. O'Brien (17/19) Sandia National Sandia National Components (17/19) Sandia National Nation											
Sc. RR-70-64 Potential EDNA Performance Characteristics R. L. Coats 05/1970 Corporation Sandia National Laboratory, Sandia National Laboratory	75.0	00 DD 00 2700			04/4067	1	fa. 1				· ·
SC.RR-70-64 Potential EDNA Performance Characteristics R. L. Coats 05/1970 Corporation None Potential EDNA Performance Characteristics R. L. Coats 05/1970 Corporation None Potential EDNA Performance Characteristics R. L. Coats 05/1970 Corporation None Potential EDNA Performance Characteristics R. L. Coats 05/1970 Corporation None Potential EDNA Performance Characteristics R. L. Coats 05/1970 Corporation None Potential EDNA Performance Characteristics R. L. Coats 05/1970 Corporation None Potential EDNA Performance Characteristics R. L. Coats 05/1970 Corporation None Potential EDNA Performance Characteristics R. L. Coats 05/1970 Corporation None Potential EDNA Performance Characteristics R. L. Coats 05/1970 Corporation None Potential EDNA Performance Characteristics R. L. Coats 05/1970 Corporation None Potential EDNA Performance Characteristics R. L. Coats 05/1970 Corporation None Potential EDNA Performance Characteristics R. L. Coats 05/1970 Corporation None Potential EDNA Performance Characteristics R. L. Coats 05/1970 Corporation None Potential EDNA Performance Characteristics R. L. Coats 05/1970 Corporation None Potential EDNA Performance Characteristics R. L. Coats 05/1970 Corporation None Potential EDNA Performance Characteristics R. L. Coats 05/1970 Corporation None Potential EDNA Performance Characteristics R. L. Coats 05/1970 Corporation None Potential EDNA Performance Characteristics Report of the Propulsion of Alzacteristics Report None Potential EDNA Performance Characteristics Report None Potential EDNA Performance	756	SC-RR-66-2706	SPR II Safety Analysis Report	R. L. Coats, P. D. O'Brien	01/1967	Corporation	[None]	experiment safety analysis	good	N	
Sc. Re.70-64   Potential EDNA Performance Characteristics   R. L. Coats   O5/1970   Corporation   None   good   None   good   None   Report, original, good						Condin Notional					
SERR-70-64   Potential EDNA Performance Characteristics   R. L. Coats   O5/1970   Corporation   Sandia National   Laboratory, Sandia Corporation   None   O6/1971   Corporation   None									ronart original		-
Electron Induced Neutron Production in R. L. Coates 06/1971 Group range of the Propulsion of Aircraft Project, Oak Ridge TN 1/4 Project, Oak Ridge T	757	CC DD 70 C4	Batantial EDNA Barfarrana Characteristics	D 1 C+-	05/4070		[NI 1		1		
Sc-RR-71-0399   Uranium Targets	/5/	3C_RR-70-04	Potential EDNA Performance Characteristics	R. L. COals	05/1970	<del></del>	[None]		good	IN	(AFPA).
SCRR-71-0399 Uranium Targets R. L. Coates 06/1971 Corporation [None] good N company associated with the report.  Fairchild Engine and Airplane Corp., Nuclear Energy for the Propulsion of Aircraft Project, Oak Airplane Corp., Nuclear Energy for the Propulsion of Aircraft Project, Oak Airplane Corp., Nuclear Energy for the Propulsion of Aircraft Project, Oak Aircraft Projec	1		Electron Induced Neutron Production in						report original		The report does not identify the site or
Fairchild Engine and Arplane Corp., Nuclear Energy for the Propulsion of Aircraft Project, Oak Serret, declassified  SERM-78  Control Rod Calibration Methods  F. W. Pressey  O3/31/1950  Ridge TN  Fairchild Engine and Aircraft Project, Oak Secret, declassified  Aircraft Project, Oak Sec	75.8	SC-PP-71-0300	1	P. I. Coates	06/1971	1	[None]		1	N	
Airplane Corp., Nuclear finergy for the Propulsion of Airplane Corp., Nuclear Finergy for the Propulsi	730	3C-III-71-0355	Oranium raigets	N. L. Coates	00/13/1				good	- IN	company associated with the report.
Nuclear Energy for the Propulsion of Aircraft Project, Oak Aircraf	1					1					Based on neutron physics and
SERM-78 Control Rod Calibration Methods F. W. Pressey 03/31/1950 Ridge TN 12/19/1963 experiment plan/design good project, clack series of the NEPA roject, Oak Airplane Corp., Nuclear Energy for the Propulsion of Airplane Corp., Nuclear Energy for the Propulsion of Aircraft Project, Oak Servet, declassified Airplane Corp., Nuclear Energy for the Propulsion of Aircraft Project, Oak Servet, declassified Airplane Corp., Nuclear Energy for the Propulsion of Aircraft Project, Oak Servet, declassified Aircraft Project, Oak Servet, declassifie	1					1					
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Fig. 12/19/1963 experiment plan/design good N project.  Fig. 12/19/1963 experiment pla	1					1	Secret, declassified		report, original.		
Fairchild Engine and Airplane Corp., Nuclear Energy for the Propulsion of Aircraft Project, Oak SERM-93  Fuel Heterogeneity and Thickness of Fuel Disks  J. F. Coneybear  J. W. Weale, M. H. McTaggart, H. Goodfellow, W. Paterson  Og/1953  Aldermaston A.W.R.E. (U.K.A.E.A.)  Operating Experience with the Zero-Energy  Criticality Committee Review of the Oak Ridge  Rotating Reflector Reactor (SORA)  AEC-ENEA Seminar on Intense Neutron Sources Session Illa The SORA Reactor: Design Status  Fairchild Engine and Airplane Corp., Nuclear Energy or the Propulsion of Aircraft Project, Oak Ridge TN  Aldermaston A.W.R.E. (U.K.A.E.A.)  ORNL, Union Carbide Corporation  AEC-ENEA Seminar on Intense Neutron Sources Session Illa The SORA Reactor: Design Status  AEC-ENEA Seminar on Intense Neutron Sources Session Illa The SORA Reactor: Design Status  Fairchild Engine and Airplane Corp., Nuclear Energy or the Propulsion of Aircraft Project, Oak Ridge TN  Aldermaston A.W.R.E. (U.K.A.E.A.)  Og/1953  Fairchild Engine and Airplane Corp., Nuclear Energy for the Propulsion of Aircraft Project, Oak Ridge TN  Aldermaston A.W.R.E. (U.K.A.E.A.)  In folder labelled "SORA Critical Experiments (Euratom)"  Internal use only. Also, not to be given public dissemination without approval  AEC-ENEA Seminar on Intense Neutron Sources Session Illa The SORA Reactor: Design Status  Giegerich, V. Raievski, W.	759	SERM-78	Control Rod Calibration Methods	F. W. Pressey	03/31/1950	1		experiment plan/design	1	N	
Airplane Corp., Nuclear Energy for the Propulsion of Aircraft Project, Oak SERM-93  Fuel Heterogeneity and Thickness of Fuel Disks  J. F. Coneybear  12/07/1950  Ridge TN  2/19/1963  Secret, declassified 12/19/1963  experiment plan/design good  N the split table assembly.  N the split table assembly.  Aldermaston A.W.R.E. (U.K.A.E.A.)  [None]  Paper submitted to IAEA Symposium on Exponential and Critical Experiments, Amsterdam. Two glossy 8&W photos of the split table assembly.  N the split table assembly.  In folder labelled "SORA Critical experiment plan/design good  N the split table assembly.  In folder labelled "SORA Critical experiments plan/design good  N the split table assembly.  In folder labelled "SORA Critical experiments plan/design good  N Experiments (Euratom)"  N the split table assembly.  In folder labelled "SORA Critical experiments plan/design good  N Experiments (Euratom)"  N the split table assembly.  In folder labelled "SORA Critical experiments plan/design good  N Experiments (Euratom)"  AEC-ENER Seminar on Intense Neutron Sources Session IIII The SORA Reactor: Design Status  Session IIII The SORA Reactor: Design Status  Session IIII The SORA Reactor: Design Status  N Experiments (Euratom)"  AL ALarrimore, R. Hass, K. Giegerich, V. Raievski, W. Giegerich, V. Raievski, W.		-		,	, , , , , , , , , , , , , , , , , , , ,			,	<u> </u>	·	
Nuclear Energy for the Propulsion of Aircraft Project, Oak Ridge TN  SERM-93  Fuel Heterogeneity and Thickness of Fuel Disks  J. F. Coneybear  J. W. Weale, M. H. McTaggart, H. Goodfellow, W. Paterson  Operating Experience with the Zero-Energy Reactor VERA  Notaggart, H. Goodfellow, W. Paterson  Operating Experience with the Zero-Energy Reactor VERA  Notaggart, H. Goodfellow, W. Paterson  Operating Experience with the Zero-Energy Reactor VERA  Notaggart, H. Goodfellow, W. Paterson  Operating Experience with the Zero-Energy Reactor VERA  Notaggart, H. Goodfellow, W. Paterson  Operating Experience with the Zero-Energy Reactor VERA  Notaggart, H. Goodfellow, W. Paterson  Operating Experience with the Zero-Energy Reactor (Sora)  Notaggart, H. Goodfellow, Reactor VERA  In folder labelled "SORA Critical Wethout approval without approval without approval safety review  Pater dealers Service Reactor (Sora)  Notaggart, H. Goodfellow, Reactor VERA  In folder labelled "SORA Critical Wethout approval without approval wi						_					
the Propulsion of Aircraft Project, Oak Aidge TN  1. W. Weale, M. H. McTagart, H. Goodfellow, W. Paterson  Operating Experience with the Zero-Energy Reactor VERA  Operating Experiment Jelon/Leasure Review of the operating In folder labelled "SORA Critical Experiments, Aidermactor VERA  In folder labelled "SORA Crit											Paper submitted to IAEA Symposium on
Aircraft Project, Oak Ridge TN  2/19/1963  SERM-93  SERM-94  SERM-94  SERM-94  SERM-94  SERM-94  SERM-95  SERM-95  SERM-93  SERM-94  SERM-94  SERM-94  SERM-94  SERM-94  SERM-95  SERM-95  SERM-93  SERM-94  SERM-	1					A Company of the Comp					
Fuel Heterogeneity and Thickness of Fuel Disks  Operating Experience with the Zero-Energy Reactor VERA  Operating Experience with the Zero-Energy No. Wagagart, H. Goodfellow, W. Paterson  Operating Experience with the Zero-Energy No. Wagagart, H. Goodfellow, W. Paterson  Operating Experience with the Zero-Energy No. Wagagart, H. Goodfellow, W. Paterson  Operating Experience with the Zero-Energy No. Wagagart, H. Goodfellow, W. Paterson  Operating Experience with the Zero-Energy No. Holder labelled "SORA Critical Experiments (Euratom)"  In folder labelled "SORA Critical Experiments (Euratom)"  AEC-ENEA Seminar on Intense Neutron Sources Session Illa The SORA Reactor: Design Status  Operating Experience with the Zero-Energy No. Wagagart, H. Goodfellow, M. Wagagart, H. Goodfellow, W. Paterson  Operating Experiment plan/design experiment plan/design experimental criticality data, experiment							Secret, declassified		report, original,		
J. W. Weale, M. H. McTaggart, H. Goodfellow, Reactor VERA  J. W. Weale, M. H. McTaggart, H. Goodfellow, W. Paterson  Ogyl953  Aldermaston A.W.R.E. (U.K.A.E.A.)  In folder labelled "SORA Critical experiment plan/design experiment plan/design  ogod  Teport, original, good  Ferent, original, good  N. In folder labelled "SORA Critical experiments (Euratom)"  Normalization of the Carlow of the Oak Ridge Rotating Reflector Reactor (SORA)  AEC-ENEA Seminar on Intense Neutron Sources Session Illa The SORA Reactor: Design Status  Segond  N. Experiments (Euratom)"  AEC-ENEA Seminar on Intense Neutron Sources Session Illa The SORA Reactor: Design Status  In folder labelled "SORA Critical experiments (Euratom)"  Aldermaston A.W.R.E. (U.K.A.E.A.)  Ogyl953  Aldermaston A.W.R.E. (U.K.A.E.A.)  Ogyl953  Aldermaston A.W.R.E. (Internal use only. Also, not to be given public dissemination without approval safety review good  N. Experiments (Euratom)"  AEC-ENEA Seminar on Intense Neutron Sources Session Illa The SORA Reactor: Design Status  In folder labelled "SORA Critical report, original, report, or	760	SERM-93	Fuel Heterogeneity and Thickness of Fuel Disks	J. F. Coneybear	12/07/1950			experiment plan/design	1	N	
Operating Experience with the Zero-Energy Reactor VERA  MCTaggart, H. Goodfellow, W. Paterson  Op/1953  Aldermaston A.W.R.E. (U.K.A.E.A.)  [None]  experimental criticality data, experiment plan/design  op od  Y Experiments (Euratom)"  In folder labelled "SORA Critical Experiments (Euratom)"  In folder labelled "SORA Critical Experiments (Euratom)"  ORNL-CF-68-4-5  Rotating Reflector Reactor (SORA)  AEC-ENEA Seminar on Intense Neutron Sources Session Illa The SORA Reactor: Design Status  AEC-ENEA Seminar on Intense Neutron Sources Session Illa The SORA Reactor: Design Status  OP/1953  Aldermaston A.W.R.E. (U.K.A.E.A.)  In folder labelled "SORA Critical experiment plan/design experiment plan/design  op od  N Experiments (Euratom)"  AEC-ENEA Seminar on Intense Neutron Sources Session Illa The SORA Reactor: Design Status  Op/1953  Aldermaston A.W.R.E. (U.K.A.E.A.)  In folder labelled "SORA Critical without approval safety review good N Experiments (Euratom)"  In folder labelled "SORA Critical report, original, or port,						T					†
761 SM 42/19 Reactor VERA W. Paterson 09/1953 (U.K.A.E.A.) [None] experiment plan/design good Y Experiments (Euratom)"    None			Operating Experience with the Zero-Energy	· ·		Aldermaston A.W.R.E		experimental criticality data,	report, original,		In folder labelled "SORA Critical
Criticality Committee Review of the Oak Ridge Rotating Reflector Reactor (SORA)  AEC-ENEA Seminar on Intense Neutron Sources Session Illa The SORA Reactor: Design Status  AEC-ENEA Seminar on Intense Neutron Sources Session Illa The SORA Reactor: Design Status  AEC-ENEA Seminar on Intense Neutron Sources Session Illa The SORA Reactor: Design Status  AEC-ENEA Seminar on Intense Neutron Sources Session Illa The SORA Reactor: Design Status  AEC-ENEA Seminar on Intense Neutron Sources Session Illa The SORA Reactor: Design Status  AEC-ENEA Seminar on Intense Neutron Sources Session Illa The SORA Reactor: Design Status  AEC-ENEA Seminar on Intense Neutron Sources Session Illa The SORA Reactor: Design Status  AEC-ENEA Seminar on Intense Neutron Sources Session Illa The SORA Reactor: Design Status  AEC-ENEA Seminar on Intense Neutron Sources Session Illa The SORA Reactor: Design Status  AEC-ENEA Seminar on Intense Neutron Sources Session Illa The SORA Reactor: Design Status  AEC-ENEA Seminar on Intense Neutron Sources Session Illa The SORA Reactor: Design Status  AEC-ENEA Seminar on Intense Neutron Sources Session Illa The SORA Reactor: Design Status	761	SM 42/19	Reactor VERA	W. Paterson	09/1953	(U.K.A.E.A.)	[None]	experiment plan/design	good	Υ	Experiments (Euratom)"
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762 ORNL-CF-68-4-5 Rotating Reflector Reactor (SORA) Staff 04/03/1968 Corporation without approval safety review good N Experiments (Euratom)"  AEC-ENEA Seminar on Intense Neutron Sources Session Illa The SORA Reactor: Design Status Giegerich, V. Raievski, W. Giegerich, V. Raievski, W.							Also, not to be given				
AEC-ENEA Seminar on Intense Neutron Sources Session Illa The SORA Reactor: Design Status Giegerich, V. Raievski, W. In folder labelled "SORA Critical"			Criticality Committee Review of the Oak Ridge			ORNL, Union Carbide	public dissemination		report, original,		In folder labelled "SORA Critical
Session Illa The SORA Reactor: Design Status Giegerich, V. Raievski, W. In folder labelled "SORA Critical	762	ORNL-CF-68-4-5			04/03/1968	Corporation	without approval	safety review	good	N	Experiments (Euratom)"
			1								
763 No report number Report Kley 09/1966 EURATOM, Ispra, Italy [None] experiment plan/design good N Experiments (Euratom)"			Session IIIa The SORA Reactor: Design Status						report, original,		
	763	No report number	Report	Kley	09/1966	EURATOM, Ispra, Italy	[None]	experiment plan/design	good	N	Experiments (Euratom)"

								report, original,		In folder labelled "SORA Critical Experiments (Euratom)". More than 45 memos regarding the ORNL SORA reactor program, to/from ORNL, AEC, and EURATOM staff, dating from 1965 to
764	No report number	The SORA Reactor	V. Raievski	03/1964	EURATOM, Ispra, Italy	[None]	experiment plan/design	good memos, originals and copies, mostly	N	1968.
765	No report number	See note	See note	See note	See note	[None]	experiment plan/design	good	N	
766	STEWS-TE-E	Report of Reactor Excursion During Test of Modified Core FBRF Operations Report No. 2	R. L. Long	07/1965	White Sands Missile Range, Army Missile Test and Evaluation Directorate	Reproduction of this report in whole or in part is prohibited except with permission of the Army Missile Test and Evaluation Directorate	criticality accident	report, original,	Y	Memo from E. Grueling to Bengt Carlson and Carson Mark. No formal report cover, report number handwritten on first page of memo.
700	STEWS-TE-E	imodified core TBM Operations Report No. 2	IX. L. LONG	07/1303	Directorate	Directorate	criticality accident	memo,	· · · · · · · · · · · · · · · · · · ·	mst page of memo.
767	T-169	Recalculation of the Critical Masses of U and Pu Water Tamped Solutions	E. Grueling	09/01/1949	Not specified	Secret, declassified 02/25/1957	computational method/data (1)	negative photostatic copy, fair	N	
			ANS-8 Standards	33, 32, 1343		, 20, 200,	1-1			
			Subcommittee, A. D.		Goodyear Atomic			report, original,		
768	TID-7016 Rev. 1	Nuclear Safety Guide	Callihan (Chair)	1961	Company	[None]	handbook/bibliography transport safety analysis,	good	N	
		Guide to Shipment of U-235 Enriched Uranium			Oak Ridge Operations Office, U. S. Atomic		equipment/process design, operating	report, original,		
769	TID-7019	Materials	H. F. Henry	1959	Energy Commission	[None]	procedures/requirements	good	N	
770	TID-7028	Critical Dimensions of Systems Containing U-235, Pu-239, and U-233	H. C. Paxton, J. T. Thomas, Dixon Callihan, and E. B. Johnson	June 1964	LASL and ORNL	[None]	Single Units & Arrays, Poisons, Reflectors, Complex Shapes, Metals & Solutions	report, original,	J-5700	145 pages, 130 references, 27 tables, 96 figures. Some normalization of geometry features.
771	TID-14844	Calculation of Distance Factors for Power and Test Reactor Sites	J. J. DiNunno, F. D. Anderson, R. E. Baker, R. L. Waterfield	03/23/1962	Division of Licensing and Regulation, U. S. Atomic Energy Commission	[None]		report, original,	N	Guidance for compliance to Code of Federal Regulations requirements. Discusses "maximum credible accident," reactor siting criteria, dose limits to the public, and estimation of potential doses to the public.
772	TID-10068	An Interpretation of Data on Enriched Hydrogenous Thermal Reactors	M. Danzker	01/09/1951	Naval Reactor Program, Westinghouse Electric Corporation, Pittsburg PA		computational method/data	report, original,	N	
		Energy Spectrum of Neutrons from Thermal		,,	Los Alamos Scientific		(-)			
		Neutron Fission of U <sup>235</sup> and From an Untamped	G. M. Freye Jr., J. H.		Laboratory (now LANL), University of	Confidential, declassified		report, original,		
773	TID-10073	Multiplying Assembly of U <sup>235</sup>	Gammel, L. Rosen	05/1954	California	02/10/1958	nuclear measurement/data	good	N	
		The Nuclear Aspects of the Accidental Criticality at Wood River Junction, Rhode Island July 24,	F. R. Nakache, M. M.		United Nuclear			report, copy from microcard,		
774	TID-21995	1964	Shapiro	11/12/1964	Corporation Division of	[None]	criticality accident	fair	Υ	
775	TID-22268	Operational Accident and Radiation Exposure Experience Within the United States Atomic Energy Commission 1943-1964	Staff	04/1965	Operational Safety, U. S. Atomic Energy Commission	[None]	operational/test/material data	report, copy from microcard,	N	
776	TRG Information Series 17	DOCUMENT MISSING FROM FOLDER	J. Bertrand, D. Breton, R.	04/ 1703	COMMISSION	[NOTE]	operational test/indice lat Udda	ruii	IN	English translation of the French report describing critical experiment facilities and programs at Saclay. Paper was presented at the O.E.E.C E.N.E.A. Symposium on Criticality Control, Karlsruhe, May 1961.
			Caizergues, C. Clouet							
777	TRG Information Series 223 (D)	The Organisation of Experimental Criticality Research	d'Orval, E. Deilgat, M. Molbert, P. Verriere	1963	U.K.A.E.A. (Risley, Lancashire)	Unclassified	experiment plan/design	report, original, good	N	

778	TNCC (UK)-54	340	S. K. Pattenden, R. B. Tattersall	08/1959	Harwell A.E.R.E. (U.K.A.E.A.)	Official use only. Not to be communicated to any person not authorized to receive it.	nuclear measurement/data	report, copy,	N	
779	UCRL-4957	Spherical and Cylindrical Plutonium Critical Masses	F. A. Kloverstorm	09/1957	Radiation Laboratory, Livermore Site (now LLNL), University of California	[None]	experimental criticality data	report, original, good	Υ	
780	UCRL-4975	DOCUMENT MISSING FROM FOLDER								
781	UCRL-4983-T	Studies of Enriched Uranium Graphite Reactor Systems	A. J. Kirschbaum	11/01/1957	Radiation Laboratory, Livermore Site (now LLNL), University of California	[None]	experimental criticality data	report, original,	Y	
782	UCRL-5255	DOCUMENT MISSING FROM FOLDER								
783	UCRL-5349	Critical Parameters of Spherical Systems of Alpha- Phase Plutonium Reflected by Beryllium	H. R. Ralston	09/10/1958	Radiation Laboratory, Livermore Site (now LLNL), University of California Lawrence Radiation Laboratory (now	[None]	experimental criticality data	report, original, good	Y	
		Hazards Summary Report The Kukla Prompt	,	/ /	LLNL), University of			report, original,		
784	UCRL-6105	Critical Assembly	E. R. Christie, B. W. Mar	02/24/1960	California Lawrence Radiation	[None]	experiment safety analysis	good	N	
785	UCRL-6504	Preliminary Results of High-Temperature Bare  U <sup>235</sup> -C Critical Assembly Measurements	R. G. Finke	06/06/1961	Lawrence Radiation Laboratory (now LLNL), University of California Lawrence Radiation	[None]	experimental criticality data	report, original,	Y	
786	UCRL-6729	Reactivity Effects of Various Reflectors on Near- Homogeneous, BeO-Moderated, Oralloy-Fueled Systems	J. R. Morton, F. J. Shon, T. F. Weirich, L. L. Gardner	01/02/1962	Laboratory (now LLNL), University of California Lawrence Radiation	[None]	experimental criticality data	report, original, good	Y	
787	UCRL-6901	Room Temperature Critical Measurements on Thorium-Loaded, Graphite-Moderated, Orally- Fueled Systems	G. M. Benson, R. H. Fox	06/30/1962	Laboratory (now LLNL), University of California	[None]	experimental criticality data, nuclear data	report, original,	Υ	
788	UCRL-6980	High-Temperature Bare BeO Critical Experiments: General Description and Preliminary Results	R. G. Finke	06/29/1962	Lawrence Radiation Laboratory (now LLNL), University of California	[None]	experimental criticality data	report, original, good	Y	
789	UCRL-7345	Health Physics Following a Nuclear Excursion: The LRL Incident of 26 March 1963	R. L. Kathren, W. C. Day, D. H. Denham, J. L. Brown	06/03/1963	Lawrence Radiation Laboratory (now LLNL), University of California	[None]	criticality accident	report, copy, fair+1779	Υ	
790	UCRL-7695	Safety Analysis Report for the Super Kukla	W. S. Gilbert, F. A. Kloverstrom, F. Rienecker Jr.	02/04/1964	Lawrence Radiation Laboratory (now LLNL), University of California	[None]	experiment safety analysis	report, original,	Υ	The report cover gives the same issue date as the initial version (02/04/1964) but an interior page identifies the revision date as 04/12/1966.
791	UCRL-7695 Rev. I	Safety Analysis Report for the Super Kukla	W. S. Gilbert, F. A. Kloverstrom, F. Rienecker Jr.	02/04/1964	Lawrence Radiation Laboratory (now LLNL), University of California	[None]	experiment safety analysis	report, original,	N	
792	UCRL-8417	Notes on Statistics for Physicists	J. Orear	08/13/1958	Lawrence Radiation Laboratory (now LLNL), University of California Lawrence Radiation	[None]	computational method/data (1)	report, original,	N	Mathematical routine for empirical fits.
793	UCRL-8523	A Practical Guide to the Method of Least Squares	P. Cziffra, M. J. Moravcsik	10/17/1958	Laboratory (now LLNL), University of California	[None]	computational method/data (3)	report, original,	N	Mathematical routine for empirical fits.

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					Lawrence Radiation					
					Laboratory (now					
					LLNL), University of		computational method/data	report, original,		
794	UCRL-8523 Rev.	A Practical Guide to the Method of Least Squares	P. Cziffra, M. J. Moravcsik	06/05/1959	California	[None]	(3)	good	N	
					Lawrence Radiation					
					Laboratory (now					
					LLNL), University of			report, original,		
795	UCRL-14174	Reconditioning of Plastic-Coated Fuel Elements	D. Freitas	05/1965	California	[None]	operational/test/material data	good	N	
					Lawrence Radiation					Benchmark models for some of the
					Laboratory (now					experiments are provided in IHECSBE
					LLNL), University of			report, original,		report PU-MET-FAST-003, -004, and -
796	UCRL-14413	The FRAN Prompt Burst Reactor	D. B. Stillman, S. W. Mead	09/29/1965	California	[None]	experiment plan/design	good	N	017.
					Lawrence Radiation			f		
			J. R. Morton III, G. A.		Laboratory (now					
		Summary Report of Critical Experiments	Pierce, L. L. Gardner. C. J.		LLNL), University of			report, original,		
797	UCRL-50175	Plutonium Array Studies, Phase I	Ball	12/22/1966	California	[None]	experimental criticality data	good	Υ	
	OCINE SOLVS	Tutomani Array Statics, Thuse I	J. R. Brown, B. H.	12/22/1500	Westinghouse Electric		experimental criticality data	Bood	<u>-</u>	
		Departur Droportios of Mater Maderated Clightly						roport original		
700	WARD 447	Reactor Properties of Water Moderated Slightly	Noordhoff, A. E. Profio, W.	11/1054	Corporation, Pittsburg			report, original,	.,	
798	WAPD-117	Enriched Uranium Lattices	O. Bateson	11/1954	PA	02/19/1958	experimental criticality data	good	Υ	
					Westinghouse Electric					
		Critical Experiments on a Highly Enriched	J. R. Brown, B. H.		Corporation, Pittsburg			report, original,		
799	WAPD-128	Homogeneous Reactor	Noordhoff, W. O. Bateson	05/1955	PA	10/06/1958	experimental criticality data	good	ΥΥ	
		Measurements of Thermal Utilization,						1		
		Resonance Escape Probability, and Fast Fission			Westinghouse Electric	Confidential,				
		Factor of Water Moderated Slightly Enriched			Corporation, Pittsburg	declassified		report, original,		Note in folder stating "John Mihalczo 8-6-
800	WAPD-134	Uranium Lattices	A. Z. Kranz	09/1955	PA	12/29/1955	experimental criticality data	good	Υ	1958 WAPD-151"
801	WAPD-151	DOCUMENT MISSING FROM FOLDER								
								1		
		Kinetic and Buckling Measurements on Lattices	J. R. Brown, D. R. Harris, F.		Westinghouse Electric					
		of Slightly Enriched Uranium and UO <sub>2</sub> Rods in	S. Frantz, J. J. Volpe, J. C.		Corporation, Pittsburg			report, original,		
802	WAPD-176	Light Water	Andrews, B. H. Noordhoff	01/1958	PA	[None]	experimental criticality data	good	Υ	
803	WAPD-185	DOCUMENT MISSING FROM FOLDER	randrews, Brin Hoording	01,1550		[HOILE]	experimental entireality data	Bood		
003	WAI D 103	DOCOMENT MISSING TROM TO EDER			Westinghouse Electric			<del>                                     </del>		
		Monte Carlo Methods and Their Application to			Corporation, Pittsburg		computational method/data	report, original,		
904	WAPD-195	1	I Coopies	07/1959	PA			1	N	
804	WAPD-195	Neutron Transport Problems	J. Spanier	07/1959	PA	[None]	(2)	good	N	
			G. G. Smith, J. W. Beck, S.							
								1		
			S. Glickstein, P. G.					1		
			Johnson, J. D. Korsmeyer,							
			P. H. Lehmann, S. Milani, J.		Westinghouse Electric					
		Annular Seed-Blanket Reactor Critical	A. Mitchell, C. D. Russell,		Corporation, Pittsburg	3		report, original,		
805	WAPD-TM-621	Experiments	S. H. Weiss, L. L. Wheat	02/1967	PA	[None]	experimental criticality data	good	Y	
										Describes method to determine total
806	WAPD-MRP-45	DOCUMENT MISSING FROM FOLDER								power generation in a fissile assembly.
					Westinghouse Electric					
		Absolute Power Calibration of a Flexible Survey			Corporation, Pittsburg	g		report, original,		
807	WAPD-TM-74	Assembly	R. N. Olcott, D. Brown	08/1957	PA	[None]	operational/test/material data	1	N	
		1		,	Westinghouse Electric		The state of the s	5		
		Nuclear Analysis of Thermal Reflected Cylindrical	G P Rutledge P A		Corporation, Pittsburg		computational method/data	report, original,		
808	WARD TM 244	1		11/1060		1		1	ķ.	
800	WAPD-TM-244	Homogeneous Critical Assemblies	Kantorczyk	11/1960	PA Wostinghouse Floatrie	[None]	(1)	good	N	
		Small Uranium-233 Fueled Seed-and-Blanket			Westinghouse Electric			1		
067		Critical Experiments (LWBR-LSBR Development		44/:	Corporation, Pittsburg			report, original,		
809	WAPD-TM-614	Program)	S. Milani, S. H. Weiss	11/1967	PA	[None]	experimental criticality data	good	Υ	
								1 1		
			G. G. Smith, J. W. Beck, S.							
			S. Glickstein, P. G.							
			Johnson, J. D. Korsmeyer,					1		
			P. H. Lehmann, S. Milani, J.		Westinghouse Electric	:				
		Annular Seed-Blanket Reactor Critical	A. Mitchell, C. D. Russell,		Corporation, Pittsburg			report, original,		
810	WAPD-TM-621	Experiments	S. H. Weiss, L. L. Wheat	02/1967	PA	[None]	experimental criticality data	good	Υ	
	5 521		Z Treiss, E. E. Triicut	02,1307	Westinghouse Electric			1000	·····	<del> </del>
		An On-Line Solid-State Reactivity Computer for	D. J. Miller, W. A.		Corporation, Pittsburg			report, original,		
811	WAPD-TM-896	1 .		00/1070			operational/test/metarial data	1	ķ.	
	188 APD-1181-090	Reactor Physics Testing	Shaughnessy	08/1970	PA	[None]	operational/test/material data	good	N	

		An Analytical Study of the Minimum Critical								
		Mass of Highly Enriched U <sub>235</sub> When Reflected by			Westinghouse Electric					
040		Natural Uranium and Water Mixtures in		02/4070	Corporation, Pittsburg		computational method/data	report, original,		
812	WAPD-TM-933	Optimized Lattices	A. W. Gray, L. L. Jones Jr.	03/1970	PA Division of	[None]	(1)	good	N	
					Production, U. S.					
					Atomic Energy		computational method/data	roport original		
813	WASH-183	An Economic Approach to Sample Size	M. N. Hudson	10/1954	Commission	[None]	(3)	report, original, good	N	
013	WA311-103		IVI. IV. ITUUSOIT	10/1334	Commission	[None]	(3)	good		
			C. K. Beck, F. P. Cowan, K.		Division of Civilian					
			W. Downes, J. A. Fleck, J.		Application, U. S.					
		Theoretical Possibilities and Consequences of	B. H. Kuper, J. McLaughlin,		Atomic Energy			report, original,		
814	WASH-740	Major Accidents in Large Nuclear Power Plants	I. Singer, M. Smith	03/1957	Commission	[None]	reactor safety	good	N	
815	WASH-1055	DOCUMENT MISSING FROM FOLDER	. 0.,					1		
					Division of Reactor					
					Development and					
					Technology, U. S.					
		Reactor Physics Efforts Required in Support of			Atomic Energy					
816	WASH-1066	the Fast Breeder Development Program		01/1966	Commission					
					Division of Reactor			1		
					Development and			1		
					Technology, U. S.					
1		Cost-Benefit Analysis of the U. S. Breeder			Atomic Energy			report, original,		
817	WASH-1126	Reactor Program		04/1969	Commission	[None]		good	N	
										Only cover/title page and tables of
										critical experiment configurations
										(Tables I, XII, and XIII) are included. A
										copy of this report is at the ORNL library
818	WCAP-1136	DOCUMENT MISSING FROM FOLDER								(252 p.)
					Westinghouse Electric					
		Reactivity and Neutron Flux Distribution Studies			Corporation, Pittsburg			report, copy,		
819	WCAP-1433	in Multi-Region Loaded Reactor Cores	W. J. Eich, W. P. Kovacik	06/1961	PA	[None]	experimental criticality data	incomplete	Υ	
		Westinghouse Atomic Power Division Reactor			Westinghouse Electric					
		Evaluation Center Reactor Operations			Corporation, Pittsburg		operating	report, original,		
820	WCAP-2020	Procedure Manual	D. F. Hanlen	05/1962	PA	[None]	procedures/requirements	good	N	
			N. A. Bostrom, I. L.		Wright Air					
			Morgan, J. T.		Development Center,					
004		Inelastic Scattering of Fast Neutrons from	Prud'Homme, P. L.	02/4050	Texas Nuclear	fac. 1	.,,,,	report, original,		
821	WADC 58-88	Nitrogen and Oxygen	Okhuysen, A. R. Sattar	02/1958	Corporation	[None]	nuclear measurement/data	good	N	
		A Critical Review of the Cross Section and Fission	G. J. Safford, W. W.		Columbia University,					
022	WACH 4022	Parameters of U <sup>235</sup> Below 1 eV		05/40/4050	Dept. of Physics, New			report, original,		
822	WASH-1022	Parameters of U Below 1 eV	Havens Jr.	05/18/1959	York City NY	[None]	nuclear measurement/data experiment safety analysis,	good	N	
					Westinghouse Electric		experiment plan/design,			
		Safety Report for the Critical Reactor Experiment	D E Hanlen D Hunter D		Corporation, Pittsburg		operating	report, original,		
823	WCAP-1316	Facility	W. Davison	10/01/1959	PA			1	N	
823	XDC-59-9-117	DOCUMENT MISSING FROM FOLDER	VV. Davisuii	10/01/1939	PA	[None]	procedures/requirements	good	N	
825	XDC-59-9-117 XDC-60-1-157	DOCUMENT MISSING FROM FOLDER			<del> </del>	<del> </del>	-	+		
826	XDC-60-3-195	DOCUMENT MISSING FROM FOLDER			<del> </del>			<del>                                     </del>		
					<del> </del>	Secret, declassified	computational method/data	report, original,		
827	Y-533	An Empirical Study of Some Critical Mass Data	C. L. Schuske, J. W. Morfitt	12/06/1949	Y-12	03/04/1958	(1)	good	N	
<del></del>	1		2323, 3	,,	† · · · · · · · · · · · · · · · · · · ·	,,	\ \ \ \ \ \ \ \ \ \ \ \ \ \ \ \ \ \ \	19.22		
			A. D. Callihan, D. F. Cronin,			Original report reissue	d			
			J. K. Fox, J. W. Morfitt, E.			as declassified with		report, copy,		
828	Y-801 (Del.)	Critical Mass Studies, Part VI	R. Rohrer, D. V. P. Williams	08/08/1951	Y-12	deletions	experimental criticality data	fair	Υ	
			,	., ., .,	<del></del>	Secret, declassified	computational method/data	report, original,	<u>-</u>	
829	Y-829	Empirical Studies of Critical Mass Data Part II	C. L. Schuske, J. W. Morfitt	12/05/1951	Y-12	03/04/1958	(1)	good	N	
	-			,,	<del></del>	Secret, declassified	computational method/data	report, original,		
830	Y-839	Empirical Studies of Critical Mass Data Part III	C. L. Schuske, J. W. Morfitt	01/16/1952	Y-12	06/21/1960	(1)	good	N	
		Application of Criticality Information to Y-12				Secret, declassified		report, original,		
831	Y-853	Plant Problems	C. L. Schuske	03/11/1952	Y-12	06/15/1960	equipment/process design	good	N	
						Secret, declassified	T	report, original,		Contains good-quality photos of
832	Y-897	Potential Hazards of Criticality Accidents	C. L. Schuske	08/05/1952	Y-12	03/04/1958	criticality accident	good	N	experimental components.
										· · · · · · · · · · · · · · · · · · ·

						Secret, declassified		report, original,		
022	V 991	A Craphita Madaratad Critical Assambly CA 4	E. L. Zimmerman	12/07/1052	V 12		aunavimental ariticality data	1	V	
833	Y-881	A Graphite Moderated Critical Assembly CA-4	E. L. Zimmerman	12/07/1952	Y-12	09/26/1957	experimental criticality data	good	Υ	
00.4	V 054	Selected Properties of 2-Furaldehyde A	5 1 6 1	04/00/4050	v 40			report, original,		
834	Y-951	Literature Search	F. L. Sachs	04/09/1953	Y-12	Unclassified	operational/test/material data	<del></del>	N	
		Minimum Critical Mass and Uniform Thermal				Secret, declassified	computational method/data	report, original,		
835	Y-1023	Neutron Core Flux in an Experimental Reactor	J. W. Morfitt	12/01/1953	Y-12	01/22/1958	(1)	good	N	
						Confidential,				
		Neutron Dose Calibration of Indium Personnel	J. W. Wachter, L. C.			declassified		report, original,		
836	Y-1092	Dosimeters for Prompt-Critical Metal Bursts	Emerson	03/01/1956	Y-12	09/11/1962	dosimetry, criticality accident	good	N	
			J. W. Wachter, B. J.							
		Response of Radiation Monitors to Prompt-	Youngblood, S. F.					report, original,		
837	Y-1182	Critical Bursts	Groothius	04/08/1958	Y-12	Unclassified	criticality accident	good	N	
838	Y-1234	DOCUMENT MISSING FROM FOLDER								
		Criticality Considerations in the Design of Plants						report, original,		
839	Y-1245	Using U <sup>235</sup>	J. D. McClendon	01/20/1959	Y-12	Unclassified	equipment/process design	good	N	
	1-1243	Using U	J. D. MICCIEIIGOII	01/20/1939	1-12	Officiassified	computational method/data	report, original,		
040	V 1210	Cuitical Accountilise of Harmitima Adotal	D. Cooler M. T. Man	02/26/4050	V 12	t to all a selfic at		1		
840	Y-1248	Critical Assemblies of Uranium Metal	R. Gwin, W. T. Mee	03/26/1959	Y-12	Unclassified	(1)	good	N	
841	Y-1273	DOCUMENT MISSING FROM FOLDER								
							operational/test/material			
		The Application of Data Processing Techniques					data, operating	report, original,		
842	Y-1371	to a Maintenance Work Control Program	J. D. Westbrook	08/07/1963	Y-12	[None]	procedures/requirements	good	N	
		Concentration of Boron or Cadmium in U-235					computational method/data	report, original,		
843	Y-1388	Solutions for K <sub>∞</sub> = 1	F. G. Welfare	06/05/1962	Y-12	Unclassified	(1)	good	N	
							transport safety analysis,			Overview of computational methods
		A Nuclearly Safe Container for Class II Shipments					computational method/data	report, original,		circa 1970 in use for Y-12 criticality
844	Y-1460	of Dry Uranium Compounds	W. T. Mee, F. G. Welfare	11/26/1963	Y-12	[None]	(2)	good	N	safety applications.
							transport safety analysis,	f		
		Analysis of Fissile Material Storage and Shipping					computational method/data	report, original,		
845	Y-1703	at the Y-12 Plant	E. C. Crume Jr.	01/08/1970	Y-12	[None]	(2)	good	N	
045	1 1703	An Eddy-Current Distance Gage for Precise Static		01/00/1370	1 12	[reone]	(2)	report, original,		
046	V 1702	Measurements		00/02/4074	V 12	[N 1		1		
846	Y-1793		J. F. Ellis	08/02/1971	Y-12	[None]	operational/test/material data		N	
0.47	V 4050	Validation Checks of the "ANISN" and "KENO"	G. R. Handley, C. M.	44 /20 /4072	v 40	ra. 1	computational method/data	report, original,		
847	Y-1858	Codes by Correlation with Experimental Data	Hopper	11/20/1972	Y-12	[None]	(2)	good	N	
		Validation of the "KENO" Code for Nuclear								
		Criticality Safety Calculations of Moderated, Low-	G. R. Handley, C. M.				computational method/data	report, copy,		
848	Y-1948	Enriched Uranium Systems	Hopper	06/13/1974	Y-12	[None]	(2)	good	N	
			D. F. Cronin, J. F. Fox, J. D.							
			McLendon, J. W. Morfitt,							
			C. L .Schuske, P. E.			Secret, declassified		report, original,		
849	Y-A2-71	Critical Mass Tests on Oralloy Machine Turnings	Wilkinson	02/29/1952	Y-12	03/27/58	experimental criticality data	good	Υ	
						Secret, declassified	computational method/data	report, original,		See ORNL-CF-65-12-42 for J. T. Thomas'
850	Y-A2-124	Critical Assemblies of Oralloy	R. Gwin, W. T. Mee	12/08/1953	Y-12	07/24/1964	(1)	good	N	report for the same events.
	<u> </u>	Trip Report: The OEEC-ENEA Symposium on	, , , , , , , , , , , , , , , , , , , ,	, ,		. , ,	<del>                                     </del>			
		Criticality Control, Karlsruhe, Germany, May 2-5,						1		
		1961 and Visits to Nuclear Installations in the						report, original,		
851	Y-A2-285	United Kingdom, May 8-12, 1961	J. W. Wachter	06/19/1961	Y-12	[None]		good	N	YAEC: Yankee Atomic Electric Company.
931	1-74-203	Office Killguotti, iviay 0-12, 1901	J. VV. VVACIILEI	00/12/1301	1-14	[ivone]		goou	IN .	TALC. Talikee Atomic Electric Company.
			P W Davison V E Grah	\A/aati	nghouse Electric					
		Two Pagion Critical Evacriments with Water	P. W. Davison, V. E. Grob,		-			ronart salainal		
0.5.5		Two-Region Critical Experiments with Water	D. F. Hanlen, R. D. Leamer,		ration, Pittsburg			report, original,		
852	YAEC-142	Moderated Slightly Enriched UO <sub>2</sub> Lattices	H. Ritz, E. Sandandrea	11/30/1959	PA	[None]	experimental criticality data	good	Υ	
		Preliminary Direct Cycle Reactor Assembly - Part				Secret, declassified		report, original,		
853	Y-B23-2	ļii	A. D. Callihan	05/21/1952	ORNL	09/12/1961	experimental criticality data	good	Υ	
		Evaluation of the M-101 and M-102 Shipping					transport safety analysis,			Document lacks a formal Y-12 cover
		Containers in Vermiculate-Filled 55-Gallon	G. R. Handley, E. C. Crume				computational method/data	report, original,		sheet. Little "analysis" or technical basis
854	Y-DD-19	Drums	Jr., W. T. Mee	08/26/1968	Y-12	[None]	(2)	good	N	is presented.
	<del></del>	<del>                                     </del>	† · · · · · · · · · · · · · · · · · · ·		···	,	1, ,		······	Contains a brief discussion of fixed
										nuclear accident dosimeters. Folder
								1		includes a "memo of conversation"
								1		
								1		between Calliahn and Roberts; this
		Criticality Safety Analysis of Fissile Material						report, copy,		memo contains additional details on the
855	Y-DD-49	Storage Areas in Building 9213	W. T. Mee	11/05/1969	Y-12	[None]	criticality safety analysis	good	N	dosimetry equipment.
			R. C. Hentchel, J. D.					report, original,		
856	Y-DD-112	Nuclear Accident Dosimeters Table of Locations	McClendon, E. Roberts Jr.	03/16/1972	Y-12	[None]	dosimetry	good	N	The source was immersed in water.
							· · · · · · · · · · · · · · · · · · ·			

										Addresses Ventalation, Fisiile Materials,
										Controls & Safety Systems, Super-
		Thermal Neutron Flux Distribution from a <sup>252</sup> Cf						report, original,		Prompt Critical, Radiation Monitors and
857	Y-DR-1	Spontaneous-Fission Neutron Source	D. W. Magnuson	10/17/1968	Y-12	[None]	nuclear measurement/data	1	N	Administrative Requirements
837	1-DK-1		D. W. Magnuson		1-12	[None]		good	IN	
050	v 55 3	OPERATIONAL SAFETY LIMITS for the OAK RIDGE	00055 61 66	11/08/1968	V 42	(2)	8 Pages of Safety Significant	report, original,		Figure 1 shows Ground Floor & Second
858	Y-DR-3	CRITICAL EXPERIMENTS FACILITY	ORCEF Staff	(Revised)	Y-12	(None)	Lmits in 7 operational areas	good	J-5700	Floor Plans & 3 Exclusion Areas
							7 pages of Exclusion Areas,			
							Access Control, Assembly			
		OAK RIDGE CRITICAL EXPERIMENTS FACILITY					Areas & Administrative	report, original,		
859	Y-DR-4	PERSONNE LACCESS CONTROL PROCEDURES	ORCEF Staff	11/08/1968	Y-12	(None)	Procedures	good	J-5700	
										Figure 1. shows the staff elements
										internal to ORCEF, their lines of
		Calculated Critical Radii for Bare Spheres of								supervision and integration into the Y-12
		Uranyl Nitrate-Nitric Acid Solutions and Uranium					computational method/data	report, original,		Technical Division & Program Review
860	Y-DR-11	Water Mixtures	D. W. Magnuson	03/15/1969	Y-12	[None]	(2)	good	N	Committees
	T DICTI	Water whiteres	D. W. Wagnason	03/13/1303	1 12	[None]	11 pages of text covering	Bood		Committees
							Philosophy of Operations,			
							Facility Organization, Periodic			
							Reviews, Fissile Material			Figures 1. & 2. show the Evacuation
		OAK RIDGE CRITICAL EXPERIMENTS FACILITY					Storage & Handling, and	report, original,		Routes for the Ground & Second Floors,
861	Y-DR-53	ADMINISTRATIVE MANUAL	ORCEF Staff	06/26/1971	Y-12	[None]	Quality Assurance	good	J-5700	resp.
							8 pages describing Policy,			
							General Information,			
							Evacuation Routes & Assembly			
		OAK RIDGE CRITICAL EXPERIMENTS FACILITY					Points, Procedures, &	report, original,		
063	V DD C0		ODCEE CA-ff	00/00/4074	V 12	[N 1		1		
862	Y-DR-68	EMERGENCY PROCEDURES	ORCEF Staff	06/09/1971	Y-12	[None]	Subsequent Actiion	good		
		Critical Three-Dimensional Arrays of Neutron-								
		Interacting Units: Part III Arrays of U(93.2)						report, original,		
863	Y-DR-83	Metal Separated by Various Materials	D. W. Magnuson	05/15/1972	Y-12	[None]	experimental criticality data	good	Υ	
		Calculated Critical Mass for Spheres of 233UO <sub>2</sub> -								
		H₂O Mixtures for Nuclear Criticality Safety					computational method/data	report, original,		
064	V DD 04	<u> </u>	D W M	00/20/4072	V 12	[N 1	1 .	1		
864	Y-DR-91	Evaluations	D. W. Magnuson	09/29/1972	Y-12	[None]	(2)	good	N	
		Limiting Critical Plutonium Concentrations for					computational method/data	report, original,		
865	Y-DR-92	Plutonium-Uranium-Water Mixtures	D. W. Magnuson	11/20/1972	Y-12	[None]	(2)	good	N	
		Calculated Criticality of Water Moderated Oxides								
		of Uranium-233, Thorium-232, and Carbon					computational method/data	report, original,		
866	Y-DR-107	Mixtures	J. T. Thomas	04/30/1973	Y-12	[None]	(2)	good	N	
		Critical Three-Dimensional Arrays of Neutron-								
		Interacting Units: Part IV Arrays of U(93.2)								
		Metal Reflected by Concrete and Arrays								
		Separated by Vermiculite and Reflected by						report, original,		
867	Y-DR-109	Polyethylene	D. W. Magnuson	04/30/1973	Y-12	[None]	experimental criticality data	good	Υ	
	1-01-103		D. W. Wagiiuson	04/30/13/3	1-12			+		
		Criticality Calculations for the U-235 Be		/		Secret, declassified	computational method/data	report, original,		
868	Y-F10-45	Experiment	G. M. Safanov	04/03/1951	Y-12	06/21/1960	(1)	good	N	
		The Contribution of the (n,2n) Reaction to the				Secret, declassified	computational method/data	report, original,		
869	Y-F10-55	Beryllium Moderated Reactor	C. B. Mills, N. M. Smith Jr.	06/05/1951	Y-12	04/12/1957	(1)	good	N	
										Scoping calculations to determine the
										inventory of materials needed, includes
		Numerical Technique for Criticality Calculations				Secret, declassified	computational method/data	report, original,		chemical analysis of some materials to
870	Y-F10-66	on Hydrogen Moderated Reactors	J. W. Webster	08/20/1951	Y-12	04/22/1957	(1)	good	N	be used.
	1			100,00,000		Secret, declassified	computational method/data	report, original,		This is a translation of a Norwegian
871	Y-F10-108	The ARE Critical Experiment	C. Mills, D. Scott	08/08/1952	ORNL	03/29/1961	·	1	N	report by same title, dated 09/06/1951.
0/1	1-L10-100	The Uranium Reactor at Kjeller and Its	C. WIIIIS, D. SCULL	00/00/1932	ORINL	03/23/1301	(1), experiment plan/design	good	íN	report by same title, dated 09/06/1951.
072	V 522 2	-	C E Colonel	04/47/5050	05***	Unalassifi I		report, original,		
872	Y-F33-2	Prospective Radioisotope Production	C. E. Saland	01/17/1952	ORNL	Unclassified		fair	N	
							operational/test/material	report, original,		
873	Y-KB-22	Tests of a Proposed Uranium Container	J. D. McClendon	12/03/1962	Y-12	[None]	data, transport safety analysis	good	N	
		Meeting of International Organization for								
		Standardization, Technical Committee 85,								
		Subcommittee 3, Working Group 5 and Trip						report, original,		
874	Y-KB-35	Report - AWRE, Aldermaston and UKAEA, Risley	J. D. McLendon	11/13/1963	Y-12	[None]	standards	good	N	
	1. 110 00	mepore ritine, radefination and older, histey		11, 13, 1303	. 12	[	transport safety analysis,	1000		
		A Nuclearly Safe Container for Class II Shires						roport origins!		
075	V KC 30	A Nuclearly Safe Container for Class II Shipments		11/25/5553	V 12	[81]	computational method/data	report, original,		
875	Y-KC-28	of Dry Uranium Compounds	W. T. Mee, F. G. Welfare	11/26/1963	Y-12	[None]	(2)	good	N	
876	Y-KC-96	DOCUMENT MISSING FROM FOLDER						1		

							transport safety analysis,			
							computational method/data			
		Evaluation of a Shipping Container for Enriched	E. C. Crume Jr., G. R.				(2), operational/test/material	report, original,		
877	Y-KC-106	Uranium Metal and Dry Compounds	Handley, R. H. Pletz	10/11/1967	Y-12	[None]	data	good	N	
							transport safety analysis,			
			E. C. Crume Jr., G. R.				computational method/data			
		Evaluation of a Shipping Container for Enriched	Handley, W. T. Mee, R. H.				(2), operational/test/material	report, original,		
878	Y-KC-109	Uranium Solution	Pletz	12/18/1967	Y-12	[None]	data	good	N	